

LICENSEE EVENT REPORT (LER)

Original Report Date 01/08/85

FACILITY NAME (1)												DOCKET NUMBER (2)				PAGE (3)			
Dresden Nuclear Power Station												0 5 0 0 0 2 3 7				1 OF 0 2			

TITLE (4)

Failure of Primary Containment Type "B" and "C" Leak Testing

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)										
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)				
											N/A					0 5 0 0 0				
1 2	1 1	8 4	8 4	-	0 2 3	-	0 1	0 4	2 5	8 5	N/A					0 5 0 0 0				

OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)				
POWER LEVEL (10)	0, 0, 0		20.402(b)		20.405(e)	50.73(a)(2)(iv)	73.71(b)
			20.405(a)(1)(i)		50.36(e)(1)	50.73(a)(2)(v)	73.71(c)
			20.405(a)(1)(ii)		50.36(e)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
			20.405(a)(1)(iii)		50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
			20.405(a)(1)(iv)	X	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)		50.73(a)(2)(iii)	50.73(a)(2)(ix)			

NAME		TELEPHONE NUMBER	
Lawrence Coyle	(X-483)	AREA CODE	
		8 1 5	9 4 2 - 2 9 2 0

[illegible]

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During the unit refueling outage, while performing DTS 1600-1 (Local Leak Rate Testing of Primary Containment Valves), the torus to condenser drain valve AO2-1599-61 leaked 121 SCFH. The total as found through leakage for type "B" and "C" testing at the time of the valve failure, exceeded the Technical Specifications limit of 493.116 SCFH. Safety significance was minimal since the other in-line isolation valve, AO2-1599-62, showed no leakage and was capable of isolating the line. Previous occurrence of a failure of type "B" and "C" leak testing was reported by R.O. 84-19 on Docket #050249.

Cause of the event was a misaligned stem. The valve stem was adjusted and the valve was retested and showed leakage well under the station's procedural limits of 31 SCFH for through leakage. Other primary containment isolation valves which exceeded the station's limits were repaired and tested to bring the total as left through leakage for type "B" and "C" testing to 367.98 SCFH; below the Technical Specification's limit of 493.116 SCFH.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 3 7	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	— 0 2 3	— 0 1	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During the unit refueling outage, while performing DTS 1600-1 (Local Leak Rate Testing of Primary Containment Valves), the torus to condenser drain valve AO2-1599-61 leaked 121 SCFH. This valve failure caused the as found through leakage for type "B" and "C" testing to exceed the Technical Specification's limit of 493.116 SCFH. After all the as found tests were completed the total leakage rate was calculated to be 1316.6 SCFH. Safety significance was minimal, in the case of valve AO2-1599-61, since the other in-line valve, AO2-1599-62, was capable of isolating the line. Safety significance was minimal in the case of the large as found leakage rate for type "B" and "C" testing, since the as found type "A" test, which is more representative of an actual containment leak, was 344.17 SCFH which is well under the Technical Specification limit of 616.39 SCFH. A previous occurrence of a failure of type "B" and "C" leak testing was reported by R.O. 84-19 on Docket #050249.

The failure of AO2-1599-61 was caused by a misaligned valve stem. The stem was adjusted and the valve retested well under the station's procedural limits of 31 SCFH for through leakage. All other valves which exhibited leakage rates greater than 31 SCFH were maintained and retested below the limit. The total as left through leakage for type "B" and "C" testing was 367.98 SCFH which is below the Technical Specification of .6 La or 493.116 SCFH.



Commonwealth Edison

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

April 26, 1985

DJS Ltr #85-455

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Updated Licensee Event Report #84-023-1, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(ii) to add corrective actions and to report the final type "B" and "C" leakage rates.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

LE22
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