

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) CRYSTAL RIVER UNIT 3										DOCKET NUMBER (2) 0 5 0 0 0 3 0 2				PAGE (3) 1 OF 0 2					
TITLE (4) Failure To Fully Inspect Steam Generator Tubes																			
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES N/A				DOCKET NUMBER(S) 0 5 0 0 0						
0	4	0	3	8	5	8	5	0	0	3	0	0	5	0	6	8	5	N/A	0 5 0 0 0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																	
6		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)					
POWER LEVEL (10)		20.405(a)(1)(i)				50.38(e)(1)				50.73(a)(2)(v)				73.71(e)					
0 0 0		20.405(a)(1)(ii)				50.38(e)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 306A)					
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)									
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)									
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)									
LICENSEE CONTACT FOR THIS LER (12)																			
NAME W. K. Bandhauer, Nuclear Safety Supervisor										TELEPHONE NUMBER AREA CODE 9 0 4 7 9 5 - 6 4 8 6									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD									
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO							

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 3, 1985, it was determined that a 1980 inservice inspection of the "B" Once Through Steam Generator had not fully examined the entire length of the generator tubes as required by Technical Specification 4.4.5.4.a.8.

The cause of the event was failure by testing personnel to recognize that complete penetration of the water entry region was required. Complete penetration was not possible due to damage of the tubes in 1978.

A proposed change to the Technical Specification has been submitted to modify the steam generator tube inspection requirements.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

CRYSTAL RIVER UNIT 3

YEAR	SEQUENTIAL NUMBER	REVISION NUMBER
0 5	0 0 0 3 0 2	8 5 — 0 0 3 — 0 0

0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

On April 3, 1985, Crystal River Unit 3 was shutdown in Mode 6 during a refueling outage. At approximately 1510, it was determined that a 1980 eddy current inservice inspection of the "B" Once Through Steam Generator (AB, HTEXCH) had not complied with Technical Specification 4.4.5.4.a.8. The eddy current probe had not transversed some steam generator tubes "from the point of entry completely to the point of exit."

Probe travel was prevented at the point of water entrance of some tubes by mechanical damage caused by the impingment of pieces of a Burnable Poison Rod Assembly on the generator upper tubesheet in February 1978. This event is further described in LER 78-017.

The cause of the event was the failure by testing personnel to recognize that complete penetration of the water entry region was required. No documentation can be found to indicate that this requirement was waived or that the appropriate persons were informed of the inability to meet the Technical Specification.

SAFETY CONSIDERATIONS

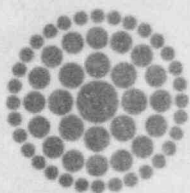
Despite the mechanical damage to some exposed tube lengths above the upper tubesheet, the eddy current test of 1980 was able to examine all tube regions that are known to be susceptible to corrosion. These regions include the tube roll area and the gap between the tubes and the tubesheet penetration. As a result, the inability to penetrate the point of water entry did not reduce the effectiveness of the eddy current examination for the previously damaged tubes.

CORRECTIVE ACTION

A proposed change to Technical Specification 4.4.5.4.a.8 has been submitted with the reload report to modify the requirement to inspect the point of water entry.

PREVIOUS SIMILAR EVENTS

This is the first occurrence of this event.



**Florida
Power**
CORPORATION

May 6, 1985
3F0585-05

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Licensee Event Report No. 85-003-00

Dear Sir:

Enclosed is Licensee Event Report (LER) No. 85-003-00 which is submitted in accordance with 10 CFR 50.73.

Should there be any questions, please contact this office.

Sincerely,

G. R. Westafer
Manager, Nuclear Operations
Licensing and Fuel Management

AEF/feb

Enclosure

cc: Dr. J. Nelson Grace
Regional Administrator, Region II
Office of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, GA 30323

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