

Duke Power Company  
Oconee Nuclear Site  
P.O. Box 1439  
Seneca, SC 29679

J. W. HAMPTON  
Vice President  
(864)885-3439 Office  
(864)885-3564 Fax



**DUKE POWER**

**DATE: August 29, 1996**

**U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555**

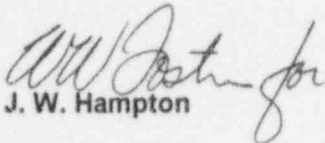
**Subject: Oconee Nuclear Station  
Docket Nos. 50-269, -270, -287  
Licensee Event Report 269/96-05**

**Gentlemen:**

**Pursuant to 10 CFR 50.73 Sections (a) (1) and (d), attached is Licensee Event Report 269/96-05, concerning a Technical Specification Surveillance Interval being exceeded.**

**This report is being submitted in accordance with 10 CFR 50.73 (a) (2) (i) (B). This event is considered to be of no significance with respect to the health and safety of the public.**

**Very truly yours,**

  
J. W. Hampton

**/fts**

**Attachment**

**cc: Mr. S.D. Ebner  
Administrator, Region II  
U.S. Nuclear Regulatory Commission  
101 Marietta St., NW, Suite 2900  
Atlanta, GA 30323**

**INPO Records Center  
700 Galleria Parkway  
Atlanta, GA 30339-5957**

**Mr. D. E. LaBarge  
U.S. Nuclear Regulatory Commission  
Office of Nuclear Reactor Regulation  
Washington, D.C. 20555**

**Mr. M. Scott  
NRC Resident Inspector  
Oconee Nuclear Station**

9609110178 960829  
PDR ADOCK 05000269  
S PDR

## LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  
Oconee Nuclear Station, Unit 1

DOCKET NUMBER (2)  
05000 269

PAGE (3)  
1 OF 5

TITLE (4)  
Technica! Specification Surveillance Interval Exceeded Due to Deficient Work Practices

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER(S)
07	30	96	96	05	00	08	29	96		05000

OPERATING MODE (9) N

POWER LEVEL (10) 100

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(v)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	<input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)	
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: L. V. Wilkie, Safety Review Manager

TELEPHONE NUMBER: (864) 885-3518

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE) X NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

On July 30, 1996, with Unit 1 at 100 % full power, a Component Engineer performing a review of mechanical snubber records discovered that the Technical Specification (TS) surveillance interval had been exceeded on a mechanical snubber. At 1030 hours, the snubber was declared inoperable and later functionally tested with acceptable results. On August 1, 1996, at 1600 hours, the snubber was declared operable. The snubber was originally installed on January 20, 1993 by a modification. The Accountable Engineer for the modification failed to ensure that the snubber had been entered into the TS surveillance maintenance procedure. The root cause for this event is Work Practices; Error Detection Practices; required verification not performed. Corrective actions include conducting a review to ensure that all snubbers have been included in the appropriate TS surveillance maintenance procedure and revising the procedure to include the missing snubber.

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)  Oconee Nuclear Station	DOCKET NUMBER (2)  05000 269	LER NUMBER (6)			PAGE (3)  2 OF 5
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
		96	05	00	

**Background**

Technical Specification 4.18 requires each snubber associated with the Reactor Coolant System and other safety related systems, as specified in the appropriate Station Procedure, to be visually inspected. It further requires that a minimum of 10% of the total snubbers in use in the plant shall be functionally tested at least once per refueling outage.

The mechanical snubber for Support/Restraint (S/R) 1-03-401B-H4171 is located on the Unit 1 Main Feedwater System piping that feeds the "B" Steam Generator (SG). The S/R is on a 24 inch diameter pipe (Class F piping) between valves 1FDW-41 (SG 1B Main Feedwater Control Valve) and 1FDW-46 (SG 1B Normal Header Check Valve).

**Description of Event**

On July 30, 1996, during a review of mechanical snubber records, a Component Engineer (CE) discovered that the mechanical snubber on Support/Restraint (S/R) 1-03-401B-H4171 was not in the appropriate maintenance inspection procedure (Snubbers-Pacific Scientific-Mechanical-Unit 1 Accessible Inspection). After further review, the CE discovered that the snubber had not been inspected since it was originally installed on January 20, 1993. Therefore, the surveillance interval in Technical Specification (TS) 4.18 was exceeded.

On July 30, 1996, at 1030 hours, the snubber was declared inoperable and preparations began to remove the snubber and perform a functional test as required by TS.

On August 1, 1996, at 1600 hours, the snubber had been functionally tested with acceptable results, reinstalled, and declared operable.

An investigation into this event was initiated. In 1989, a design error was discovered in a seismic calculation for feedwater piping. An operability evaluation was performed and it was determined that the associated piping and supports did not fully comply with the applicable code requirements, but could perform its intended function and was

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)  Oconee Nuclear Station	DOCKET NUMBER (2)  05000 269	LER NUMBER (6)			PAGE (3)  3 OF 5
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
		96	05	00	

operable. A "Station Problem Report" was initiated requesting an upgrade to meet the applicable code and a minor modification was designed in 1991. During the Unit 1 Refueling Outage in 1993, this S/R was installed. The Accountable Engineer (AE) had signed documentation indicating that all applicable procedures had been revised, but the new snubber was not added to the surveillance procedure.

An interview was conducted with the AE involved in this event. The AE indicated that he was aware of his responsibility to ensure that all procedures had been revised, but failed to ensure that the appropriate maintenance procedure was revised to incorporate the snubber. Due to the time elapsed since this omission, the AE could not recall any specific details which may have further explained this event. A review of minor modifications indicated that the AE had performed only two modifications on S/Rs. The other modification did not require a TS surveillance.

## Conclusion

The root cause of this event is Work Practices; Error Detection Practices; required verification not performed. The Accountable Engineer did not ensure that all appropriate procedures were revised as a result of the modification, specifically the Maintenance Procedure (Snubbers-Pacific Scientific-Mechanical-Unit 1 Accessible Inspection) was not revised to incorporate the snubber into the surveillance program.

Oconee has introduced several initiatives to decrease work practice errors since 1993. One of these is a Human Error Reduction Training Course, which the individual associated with this error attended in 1995. Another is a six part communication on "Improving Human Performance Results". This initiative began in July 1996 and is to be used by all site supervisors and managers to re-communicate both management's expectations for using the error reduction tools and to provide an additional opportunity to make sure everyone understands how the tools must be used.



**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Oconee Nuclear Station	05000 269	96	05	00	4 OF 5

A historical search of reportable events over the last two years indicated that no events have occurred involving Technical Specification surveillance intervals being exceeded. Therefore, this event is considered to be non-recurring.

There were no equipment failures associated with this event.

No personnel injuries, radiation exposures, or releases of radioactive materials occurred with this event.

**CORRECTIVE ACTIONS****Immediate**

1. The snubber was removed, functionally tested with acceptable results, and reinstalled.

**Subsequent**

1. Maintenance Procedure MP/1/A/3018/020 (Snubbers-Pacific Scientific-Mechanical-Unit 1 Accessible Inspection) has been revised to add the mechanical snubber for Support/Restraint 1-03-401B-H4171.

**Planned**

1. The snubber installation procedure MP/0/A/3018/058 (Snubbers-Pacific Scientific-Mechanical-Removal and Installation) will be revised to require a signoff for the Snubber Component Engineer to ensure that new snubbers are added to the appropriate Maintenance Procedure.
2. A review will be conducted to ensure that all snubbers have been included in the appropriate maintenance procedures.

Planned corrective actions 1 and 2 are considered commitments to the NRC.

NRC FORM 300A		U.S. NUCLEAR REGULATORY COMMISSION(4-95)		APPROVED OMB NO. 3150-0104 EXPIRES 4/30/98	
<b>LICENSEE EVENT REPORT (LER)</b> <b>TEXT CONTINUATION</b>				ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNSB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503	
FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)	
Oconee Nuclear Station		05000 269		YEAR 96	SEQUENTIAL NUMBER 05
				REVISION NUMBER 00	PAGE (3) 5 OF 5

### SAFETY ANALYSIS

This event involved a missed Technical Specification surveillance on a Main Feedwater System mechanical snubber. The snubber was removed, functionally tested with acceptable results, and reinstalled. Thus, the snubber would have performed its intended function during a seismic event. Even if this snubber had failed its functional test, a previous evaluation concluded that the remaining piping and supports were adequate, and considered to be operable, without the presence of the snubber. Therefore, this event had no impact on the health and safety of the public.