NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Browns Ferry - Unit 1	0 5 0 0 0 2 5 9	1 OF 0 2

TITLE (4)
Primary Containment Isolation System Initiation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									Browns Ferry - Unit 2	0 5 0 0 0 2 6 0
0 4	1 1	8 5	8 5	- 0 1 1	- 0 0	0 5		8 5	Browns Ferry - Unit 3	0 5 0 0 0 2 9 6

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
R. C. Steele	2 0 5 — 7 2 9 - 3 5 8 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
AR	J M	R L Y	G 0 8 0	Yes							
AY	J M	F U	M 1 7 5	No							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The coil in relay 16AK72 failed causing the control circuit fuse to blow. The failure caused a partial initiation of primary containment isolation system. The reactor zone isolation system, refueling zone isolation system, and standby gas treatment system were affected by the isolation.

The failed coil was replaced, and a new fuse was installed. The system was returned to normal.

This is considered to be a random failure and no further recurrence control is required.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Browns Ferry - Units 1, 2, and 3	0 5 0 0 0 2 5 9	8 5	— 0 1 1	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Unit 1 was in cold shutdown, unit 2 was in a refueling outage, and unit 3 was in cold shutdown.

On April 11, 1985, electrical coil (CL) in relay (RLY) 16AK72 (GE type CR120A) failed resulting in a control circuit fuse (FU) to blow. The event initiated a control room annunciation when unit 2 reactor building ventilation isolation occurred. The licensed operator immediately verified that no signal was present before trying to clear the annunciation. The attempt to reset the isolation signal failed.

The coil failure resulted in hydrogen oxygen analyzer B to isolate, drywell detection outboard valves FCV-90-255 and FCV-90-257A to close, and control room emergency ventilation (VI) system to initiate.

The blown fuse resulted in partial initiation of secondary containment isolation division II, i.e. initiation of reactor zone isolation (VA), refueling zone isolation (VA) and standby gas treatment (BH) system initiation. The safety systems performed as designed.

The failed relay coil was replaced and a new fuse installed. The circuit was returned to normal and the annunciation alarm was reset.

There are approximately 1300 CR120A relays installed at Browns Ferry and less than 0.4 percent per year fail. Since the failure rate is so low, these failures are considered random and no recurrence control is required. This event is not considered to be Part 21 reportable.

There was no danger to the health or safety of the public, plant employees, or equipment at any time during the event.

Responsible Plant Section - N/A

Previous Events - BFRO-50-259/8034, /8163, /8248, /8257, / 8423
BFRO-50-260/7918, /8117, /8225
BFRO-50-296/8009

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

May 10, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 1 - DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE REPORT BFRO-50-259/85011

The enclosed report provides details concerning primary containment isolation system initiation. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

JE Swindell

for G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosures
cc (Enclosures):
Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
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Atlanta, Georgia 30303

NRC Resident Inspector, BFN

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