

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) OYSTER CREEK, UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 2 1 9 1 OF 0 4										PAGE (3) 1 OF 0 4																														
TITLE (4) 4160 V EMERGENCY BUS TECHNICAL SPECIFICATION VIOLATION																																																		
EVENT DATE (5)									LER NUMBER (6)									REPORT DATE (7)									OTHER FACILITIES INVOLVED (8)																							
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES												DOCKET NUMBER(S)											
0 4			0 1			8 5			8 5			0 0 8			0 0 0 4			3 0 8			5															0 5 0 0 0														
0 4			0 1			8 5			8 5			0 0 8			0 0 0 4			3 0 8			5															0 5 0 0 0														
OPERATING MODE (9) N										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																																								
POWER LEVEL (10) 0 9 5										20.402(b)										20.406(e)										50.73(a)(2)(iv)										73.71(b)										
										20.406(a)(1)(i)										50.36(a)(1)										50.73(a)(2)(v)										73.71(a)										
										20.406(c)(1)(ii)										50.36(a)(2)										50.73(a)(2)(vi)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)										
										20.406(a)(1)(iii)										X 50.73(a)(2)(i)										50.73(a)(2)(vii)(A)																				
										20.402(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(viii)(B)																				
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LICENSEE CONTACT FOR THIS LER (12)																																																		
NAME Arthur Dickinson, Plant Engineering															TELEPHONE NUMBER 6 0 9 9 7 1 1 - 4 6 2 1 6																																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																		
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NPROS			CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NPROS																							
SUPPLEMENTAL REPORT EXPECTED (14)																				EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR																				
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO																														

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A Plant Engineering review of Technical Specification Amendment 80 found that existing procedures did not meet the new Technical Specification requirements and calibration tolerances. Existing calibration documentation was reviewed for degraded voltage relays and the degraded voltage relay timers. Although they were found to be within the acceptable tolerances stated in the existing procedures, the procedures had not been revised to incorporate the recently issued Technical Specification requirements. The Amendment was effective on the date of issuance and did not provide for an implementation period in which to revise the procedures. Immediate action was taken to temporarily change the procedures required to ensure compliance with the Technical Specification Amendment.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 368A's) (17)

DATE OF OCCURRENCE

The discrepancy was discovered on April 1, 1985.

IDENTIFICATION OF OCCURRENCE

Technical Specification Change Request No. 88 was submitted on August 11, 1980 and supplemented on October 18, 1982, December 5, 1983, February 9, 1984 and March 23, 1984. Technical Specification Amendment 80 was effective the date of issuance, February 11, 1985 and was received at the plant on February 21, 1985 with no implementation period provided to allow for the necessary procedure revisions. A review of Technical Specification Amendment 80 by Plant Engineering identified new testing and calibration requirements for the 4160 Volt Emergency Bus Undervoltage and Degraded Voltage Equipment that had not yet been incorporated in plant procedures. Therefore, the equipment was in compliance with the requirements of the existing procedures, but the procedures did not reflect the new Technical Specification requirements. This is considered reportable under 10CFR50.73a(2)i B.

CONDITIONS PRIOR TO OCCURRENCE

The plant was on line at 1825 MWt (95 percent power).

DESCRIPTION OF OCCURRENCE

The review of Technical Specification Amendment No. 80 by Plant Engineering on April 1, 1985, found that the loss of voltage relays and the degraded voltage relay were not being surveilled as required by the amended Technical Specification. The previous calibration of the degraded voltage relays in September 1984, left the "C" emergency bus out of the new  $\pm 1\%$  tolerance but within the 5% tolerance required by the procedure. The degraded voltage delay timers had not been calibrated since installation. The timers had been tested monthly since installation for greater than ten (10) second operation.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

At the time Technical Specification Amendment 80 was received at Oyster Creek on February 21, 1985, plant procedures had not been revised to ensure compliance with the new requirements. Immediate action was taken to make temporary changes to the procedures required to bring the equipment into compliance with Technical Specification Amendment 80. The degraded voltage relay set points were adjusted to the +1% tolerance and reinstalled. The time delay relays were timed and found to be running longer than ten (10) seconds. The longest time was found to be 12.31 seconds. All six (6) delay timers (three on each bus) were adjusted to 10 seconds +1 second. The plant remained on line and only one channel of undervoltage protection was disabled at a time. The undervoltage and degraded voltage relays had been calibrated to +5% on September 26, 1984 for "C" bus and on October 2, 1984 for "D" bus prior to plant startup. The condition of these relays was documented and acceptable at that time.

APPARENT CAUSE OF OCCURRENCE

The cause of the occurrence was due to a breakdown in the administrative controls which are intended to assure that the necessary actions required to implement changes in Technical Specifications are properly assigned and monitored for completion. These circuits were modified and installed in 1980, and the Technical Specification Change Request was submitted in August of 1980. This was prior to the institution of our existing Licensing Action Item (LAI) Tracking System, therefore, there was no system in place to track the change request or the responsibility to revise the necessary procedures.

In addition, no implementation period was provided and the Technical Specification Amendment was effective February 11, 1985, the date of issuance. Therefore, the procedures required to ensure compliance with the Technical Specification Amendment had not yet been revised by Plant Engineering.

ANALYSIS OF OCCURRENCE and SAFETY ASSESSMENT

The safety significance of this event is considered to be minimal. The degraded voltage relaying was set on the conservative side of tolerances. This increases the minimum operating voltage which would be connected to the electrical equipment prior to a low voltage trip. The main objective is to keep the bus energized until the degraded voltage could begin to damage equipment, the higher trip voltage should result in less or no damage to equipment.

The degraded voltage delay timers being set too long does allow for some increased motor operating currents and temperatures. The additional (approximately) 2 seconds is not considered to be a sufficient amount of time for a significant increased level of damage to occur as a result of the current or heating increase.

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TEXT (If more space is required, use additional NRC Form 388A's) (17)

CORRECTIVE ACTION

Immediate action was taken to temporarily change the procedures required to ensure compliance with the Technical Specification Amendment. Other corrective actions included counseling the responsible personnel in the actions required to ensure compliance with recent Technical Specification Amendments. In addition, future Technical Specification Change Requests will include a provision for a suitable implementation period.

(0943A)



**GPU Nuclear Corporation**

Post Office Box 388  
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Forked River, New Jersey 08731-0388  
609 971-4000  
Writer's Direct Dial Number:

April 30, 1985

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Licensee Event Report

This letter forwards one (1) copy of Licensee Event Report (LER)  
No. 85-008.

Very truly yours,

A handwritten signature in black ink, appearing to read "P. B. Fiedler", written over a horizontal line.

Peter B. Fiedler  
Vice President and Director  
Oyster Creek

PBF:KB:dam(0943A)  
Enclosures

cc: Dr. Thomas E. Murley, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station  
Forked River, NJ 08731

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