

 **Nuclear**

GPU Nuclear Corporation

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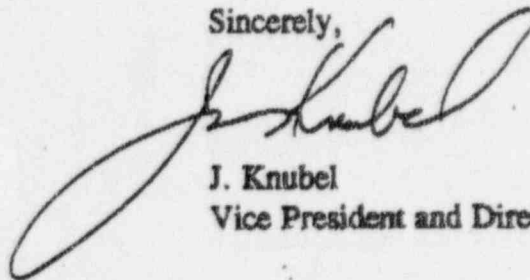
U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for March 1996

Enclosed are two copies of the March 1996 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,



J. Knubel

Vice President and Director, TMI

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WGH

Attachment

cc: Administrator, Region I
TMI Senior Resident Inspector
96001

OPERATIONS SUMMARY

March 1996

The plant entered the month operating at 100% power and remained at that power level for the remainder of the month. Net unit electrical output averaged approximately 819 MWe during March.

MAJOR SAFETY RELATED MAINTENANCE

The following major safety related maintenance item was accomplished during the month:

Auxiliary and Fuel Handling Building Filters AH-F-2A/C

The Auxiliary and Fuel Handling Building charcoal filters were replaced in filtration units AH-F-2A and AH-F-2C. In addition, the 72 HEPA filters were replaced in AH-F-2A. Following completion of the work, the units were tested and found to perform satisfactorily. AH-F-2A and C were returned to service.

OPERATING DATA REPORT

OPERATING STATUS

DOCKET NO. 50-289
 DATE
 COMPLETED BY W C HEYSEK
 TELEPHONE (717) 948-8191

1. UNIT NAME: THREE MILE ISLAND UNIT 1
 2. REPORTING PERIOD: MARCH 1996
 3. LICENSED THERMAL POWER: 2568
 4. NAMEPLATE RATING (GROSS MWe): 872
 5. DESIGN ELECTRICAL RATING (NET MWe): 819
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):

10. REASONS FOR RESTRICTIONS, IF ANY:

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	2184.0	189169.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	2184.0	111727.1
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	2184.0	110565.3
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1909976	5604198	271719124
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	644869	1894350	91347824
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	609344	1789496	85832900
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	58.4
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	58.4
21. UNIT CAPACITY FACTOR (USING MDC NET)		104.2	104.2	57.7
22. UNIT CAPACITY FACTOR (USING DER NET)		100.0	100.0	55.4
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	35.4
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: MARCH

DAY AVERAGE DAILY POWER LEVEL
(MWe-NET)

1	823
2	822
3	823
4	823
5	818
6	817
7	821
8	824
9	821
10	818
11	819
12	820
13	819
14	815
15	811
16	819

DAY AVERAGE DAILY POWER LEVEL
(MWe-NET)

17	821
18	817
19	814
20	819
21	819
22	821
23	820
24	820
25	812
26	817
27	822
28	822
29	821
30	818
31	815

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191

REPORT MONTH March 1996

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report ⁴	System Code ^{5,6}	Component Code ^{7,8}	Cause & Corrective Action to Prevent Recurrence
						None			

- 1
 F Forced
 S Scheduled
- 2
 Reason
 A Equipment Failure (Explain)
 B Maintenance or Test
 C Refueling
 D Regulatory Restrictions
 E Operator Training & Licensing Examination
 F Administrative
 G Operational Error (Explain)
 H Other (Explain)
- 3
 Method
 1 Manual
 2 Manual Scram
 3 Automatic Scram
 4 Other (Explain)
- 4
 Exhibit G - Instructions for preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)
- 5 Exhibit 1 same source
- 6 Actually used exhibits F & H NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 5, 1997**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **Yes. To support GPU Nuclear plans to do independent reload analyses for Cycle 12 as discussed in response to question 6 below, T.S. 6.5.9.2 would require revision to include references to the GPU Nuclear analysis methods applied to the reload consistent with the guidance of Draft Generic Letter (GL) 83-11.**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **A Technical Specification Change Request for the changes as discussed above would be submitted after formal issuance of GL 83-11.**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: **GPU Nuclear Letter 6710-96-2092, dated March 28, 1996 confirmed plans to perform independent reload design evaluations for Cycle 12, the next operation cycle, based on previously approved vendor methods described in GPU Nuclear Topical Reports TR-091 (core physics), TR-078 (core thermal hydraulics), TR-087 (FSAR safety analyses) and TR-092P (design and setpoints methodology) submitted to the NRC. As submitted in the March 28 letter, , Cycle 12 independent reload design activities and application results anticipate the use of formally issued NRC Draft Generic Letter 83-11, Supplement 1, "Licensee Qualification for Performing Safety Analyses". Per the Draft GL 83-11 guidance, the GPU Nuclear Cycle 12 reload program and results are expected to be available for NRC review in the March to April 1997 time frame. It is expected that the Cycle 12 results will be licensed in accordance with the requirements of 10 CFR 50.59 safety evaluations for changes.**
7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: **(a) 177 (b) 683**

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained. Phase II is expected to be started in 2002.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.