

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT II										DOCKET NUMBER (2) 0 5 0 0 0 3 6 6 1				PAGE (3) OF 0 2		
TITLE (4) FULL REACTOR SCRAM FROM SRM HI-HI																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
													0 5 0 0 0			
0 4	1 1	8 5	8 5	0 1 3	0 0 0	5 0	2 8	5					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)														
5		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				<input checked="" type="checkbox"/> 50.73(a)(2)(v)				73.71(c)		
0 0 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text: NRC Form 366A)		
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME										TELEPHONE NUMBER						
Steven B. Tipps, Superintendent of Regulatory Compliance										AREA CODE						
										9 1 2		3 6 7 1 7 8 5 1				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		
YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO						

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At approximately 1200 CST on 04/11/85, with the reactor in cold shutdown for refueling, operation personnel received the following indications: "REACTOR AUTO SCRAM" and "REACTOR NEUTRON MONITORING SYSTEM".

Following an investigation, plant personnel determined that a full reactor protection system (RPS) logic actuation had occurred (no scram since all control rods were already fully inserted) because of a Hi-Hi signal from source range monitors (SRM) 2C51-K600A, 2C51-K600C, and 2C51-K600D (components of the neutron monitoring system).

The Hi-Hi signal from the SRMs was caused by electrical noise.

Plant personnel immediately reset the RPS logic. They then confirmed proper operation of the SRMs per the "SRM FLUX RESPONSE CHECK" procedure (HNP-2-9301).

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT II	0500036685	01	3	0	0	2	OF 02

TEXT (If more space is required, use additional NRC Form 365A's) (17)

This 30 day LER is reportable per 10CFR50.73(a)(2)(iv) because this event shows that an unplanned automatic actuation of an engineered safety feature (the reactor protection system) occurred.

At approximately 1200 CST on 04/11/85, with the reactor mode switch in the refuel position, operations personnel received the following indications: "REACTOR AUTO SCRAM" and "REACTOR NEUTRON MONITORING SYSTEM TRIP".

Following an investigation, plant personnel determined that a full reactor protection system (RPS) logic actuation had occurred (no scram occurred, since all control rods were already fully inserted) because of a Hi-Hi signal from source range monitors (SRM) 2C51-K600A, 2C51-K600C, and 2C51-K600D (components of the neutron monitoring system).

The Hi-Hi signal from the SRMs was caused by electrical noise.

After the event, plant personnel immediately reset the RPS logic. They then confirmed proper operation of the SRMs by performing the "SRM FLUX RESPONSE CHECK" procedure (HNP-2-9301) at approximately 1310 CST on 04/11/85.

When the event occurred all control rods were in their inserted position; consequently, the unplanned reactor scram had no actual or potential safety consequences, nor did this event affect the health and safety of the public.

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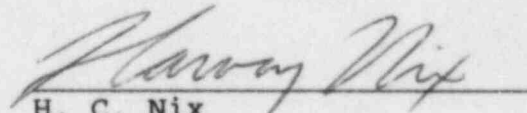
Edwin I. Hatch Nuclear Plant


May 2, 1985  
GM-85-412

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-013. This report is required by 10CFR 50.73(a)(2)(v).

  
H. C. Nix  
General Manager

  
HCN/STB/vlz

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