NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Virgil C. Summer Nuclear Station	0 5 0 0 0 3 9 5	1 OF 0 2

TITLE (4)
Inadequate Containment Penetration Protection Devices

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 3	2 5	8 5	8 5	0 0 9	0 0	0 4	3 0	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	Part 21
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
A. M. Paglia, Manager, Nuclear Licensing	8 0 3 7 4 8 - 3 9 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
B	N H			N							

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During the course of a re-evaluation of South Carolina Electric & Gas Company's (SCE&G) compliance with Regulatory Guide 1.63, "Electric Penetration Assemblies in Containment Structures for Light Water-Cooled Nuclear Power Plants," Gilbert Commonwealth, Inc. (G/C) identified that the sizing of the primary and backup overcurrent penetration protection devices for the incore instrumentation flux mapping space heaters was inadequate. SCE&G verified the inadequacy of the overcurrent protection for those circuits and, in addition, determined this condition to be a potential substantial safety hazard.

For immediate corrective action, the Licensee tagged the primary overcurrent protection device (circuit breaker) in the open position with the restriction that the space heaters can only be energized when the Plant is shutdown and containment is opened. No nuclear safety related circuits are affected; therefore, the operation and safe shutdown capability of the Plant is not impaired. However, a failure of the penetration conductor could have resulted in a breech in containment and, subsequently, a radioactive release greater than that allowed by 10CFR Part 100. The Licensee is evaluating the feasibility of modifying the protection devices to allow use of space heaters during normal plant operations.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Due to errors discovered in the Virgil C. Summer Nuclear Station Technical Specifications Table 3.8-1, "Containment Penetration Conductor Overcurrent Protective Device Test Setpoint Criteria," Nuclear Operations requested a comprehensive review of South Carolina Electric & Gas Company's (SCE&G) compliance with Regulatory Guide 1.63, "Electric Penetration Assemblies in Containment Structures for Light Water-Cooled Nuclear Power Plants." During the course of this review, Gilbert Commonwealth, Inc. (G/C) identified that the sizing of the primary and backup overcurrent penetration protection for the incore instrumentation flux mapping space heater circuits was inadequate. On March 23, 1985 during review of the G/C submittal, SCE&G verified the inadequacy of the overcurrent protection for those circuits and, in addition, determined this condition to be a potential substantial safety hazard.

Although the incore instrumentation flux mapping space heaters are normally de-energized, for immediate corrective action the Licensee tagged the primary overcurrent protection device (circuit breaker) in the open position with the restriction that the space heaters can only be energized when the Plant is shutdown and containment is opened. This condition was identified as a non-conformance to ensure that the specified corrective measures were implemented. The inadequate penetration protection was identified as a potential substantial safety hazard, and an evaluation was started to determine if this condition warranted 10CFR Part 21 reportability.

The 10CFR Part 21 evaluation revealed that no nuclear safety related circuits are affected by this non-conforming condition. Therefore, the operation and safe shutdown capability of the Plant is not impaired. However, a failure of the penetration conductor could have resulted in a breech in containment and, subsequently, a radioactive release greater than the allowed by 10CFR Part 100. The inadequate penetration protection is a substantial safety hazard and is, therefore, reportable under 10CFR Part 21.

The Licensee is evaluating the feasibility of modifying the primary and backup penetration protection devices to allow the use of space heaters during normal Plant operation.

SOUTH CAROLINA ELECTRIC & GAS COMPANY
POST OFFICE 764
COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

April 30, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Subject: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 85-009
P-21-85-001

Dear Sir:

Attached is Licensee Event Report #85-009 for the Virgil C. Summer Nuclear Station. This Report is submitted pursuant to the requirements of 10CFR21.

Should there be any questions, please call us at your convenience.

Very truly yours,

O. W. Dixon, Jr.

MDB:OWD/tdh

Attachment

cc: V. C. Summer
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