

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0104
EXPIRES - 8/31/85

FACILITY NAME (1)

Limerick Generating Station - Unit 1

DOCKET NUMBER (2)

050003521 OF 013

PAGE (3)

TITLE (4)

Automatic Closure of RHR Shutdown Cooling Inboard Suction Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)		
03	28	85	85	038	000	04	23	85		050003521		
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
4			<input checked="" type="checkbox"/> 20.402(b) <input type="checkbox"/> 20.406(a) <input checked="" type="checkbox"/> 90.73(a)(2)(iv) <input type="checkbox"/> 73.71(b)									
POWER LEVEL (10)			<input type="checkbox"/> 20.406(a)(1)(i) <input type="checkbox"/> 90.73(a)(1) <input type="checkbox"/> 90.73(a)(2)(v) <input type="checkbox"/> 73.71(a)									
000			<input type="checkbox"/> 20.406(a)(1)(ii) <input type="checkbox"/> 90.73(a)(2) <input type="checkbox"/> 90.73(a)(2)(vi) <input type="checkbox"/> OTHER (Specify in Abstract below and in Test, NRC Form 366A)									
			<input type="checkbox"/> 20.406(a)(1)(iii) <input type="checkbox"/> 90.73(a)(2)(i) <input type="checkbox"/> 90.73(a)(2)(vii)(A)									
			<input type="checkbox"/> 20.406(a)(1)(iv) <input type="checkbox"/> 90.73(a)(2)(ii) <input type="checkbox"/> 90.73(a)(2)(vii)(B)									
			<input type="checkbox"/> 20.406(a)(1)(v) <input type="checkbox"/> 90.73(a)(2)(iii) <input type="checkbox"/> 90.73(a)(2)(ix)									

LICENSEE CONTACT FOR THIS LER (12)

NAME

John C. Nagle, Engineer - Special Projects

TELEPHONE NUMBER

AREA CODE

215841-5184

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 85-038

On March 28, 1985, with Unit 1 in cold shutdown and reactor water at 135 degrees Fahrenheit, the inboard RHR shutdown cooling suction valve received an isolation signal. The valve, HV-51-1F009, closed properly to the isolation position. Cause of the isolation was failure of a procedure to provide adequate isolation reset instructions. After the isolation logic was properly reset, shutdown cooling was returned to operation.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 3 8	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Description of the Event:

On March 28, 1985, at 7:35 p.m. with Unit 1 in cold shutdown, the inboard RHR shutdown cooling suction valve HV-51-1F009 closed to the isolation position. The isolation signal was reset and shutdown cooling was returned to normal operation.

Consequences of the Event:

Shutdown cooling, which was in service at the time of the event, isolated properly. Shutdown cooling was isolated for approximately 12 minutes. There were no adverse effects.

Cause of the Event:

Cause of the event was a procedural inadequacy. At the time of the event, ST-2-036-631-1 (Instrument Line Backfill Procedure) was being performed. At the beginning of the procedure, power is removed from HV-51-1F009 so that the valve will not close when an isolation signal is generated during instrument line backfilling. At the conclusion of the backfilling, the isolation reset switch is pushed to reset the isolation signal. However, prior to pushing the reset switch, certain valve control switches must be in the 'CLOSED' position to enable the reset relay to be energized. The procedure did not require placing the appropriate valve control switches in the 'CLOSED' position. Therefore, when the reset switch was pushed, the reset relay did not energize to reset the isolation. As a result, the isolation signal remained sealed-in and HV-51-1F009 closed to the isolation position when its power was restored.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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EXPIRES 8/31/85

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TEXT (If more space is required, use additional NRC Form 366a) (1-7)

Corrective Actions:

To reset the isolation signal, the operators referred to GP-8 (Primary and Secondary Containment Isolation Verification and Reset). The appropriate valve control switches were placed in the 'CLOSED' position and the isolation logic was then reset. Shutdown cooling was immediately returned to operation.

ST-2-036-631-1 will be revised to require installation of a jumper that will simulate closure of the appropriate valve control switches. This will enable the logic to be reset without having to close fourteen valves to reset the logic. Removal of the jumper after the logic is reset will be controlled by the procedure.

Previous Similar Occurrences:

None.

Although there have been previous occurrences concerning instrument valving errors and inadequate procedures, there have been no previous similar occurrences concerning failure of a procedure to provide adequate instructions to reset an isolation.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

April 23, 1985

Docket No. 50-352

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Washington, DC 20555

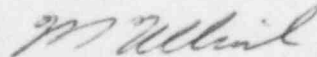
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER concerns the automatic closure of the RHR shutdown cooling inboard suction valve.

Reference:	Docket No. 50-352
Report Number:	85-038
Revision Number:	00
Event Date:	March 28, 1985
Report Date:	April 23, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

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J. T. Wiggins, Senior Site Inspector
See Service List

IE22
11

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Atomic Safety & Licensing Board Panel
Docket & Service Section (3 Copies)
James Wiggins
Timothy R. S. Campbell

January 16, 1985