

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 2 4				PAGE (3) 1 OF 01								
TITLE (4) Primary Containment Group 3 Isolation Due to Erroneous Reactor Water Cleanup System Leak High Signal																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)									
0	4	0	9	8	5	8	5	0	0	7	0	0	5	0	2	8	5	0	5	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																				
2		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)								
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)								
0		0				0				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)								
		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)(A)												
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(B)												
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(ix)												
		20.405(a)(1)(v)				50.73(a)(2)(iii)																
LICENSEE CONTACT FOR THIS LER (12)																						
NAME M. J. Pastva, Jr., Regulatory Technician										TELEPHONE NUMBER 9 1 9 4 5 7 - 2 3 1 5												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS												
X	C/E	P/S	X B	4	5	0	No															
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		
YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 4-9-85, at 0335, the Unit 2 Reactor Water Cleanup (RWC) System inlet primary containment inboard and outboard isolation valves, 2-G31-F001 and F004, automatically closed due to a primary containment Group 3 isolation signal. The Unit 2 Control Operator became aware of this event through appropriate Control Room alarm annunciation of an RWC System Leak Hi-Hi condition. Unit 2 was in startup at 3 percent power. RWC filter/demineralizer (F/D) A reject flow to the unit main condenser was being secured to prepare for establishing RWC System return flow to the reactor.

The isolation signal is attributed to F/D A reject flow oscillations which are believed to have resulted from a weld leak on the low side sensing leg of the F/D A outlet pressure breakdown orifice, 2-G31-Z002-FE-86. The flow oscillations occurred with sufficient magnitude to actuate the RWC System differential flow switch 2-G31-FDS-N603 1A or 1B, thereby resulting in the isolation signal.

The subject weld leak was repaired and the RWC System was returned to service.

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Carolina Power & Light Company

Brunswick Steam Electric Plant
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May 2, 1985

FILE: B09-13510C
SERIAL: BSEP/85-0834

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 2
DOCKET NO. 50-324
LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-85-007

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/clh

Enclosure

cc: Dr. J. N. Grace

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