

# LICENSEE EVENT REPORT (LER)

Facility Name (1) <div style="text-align: center;">Byron, Unit 1</div>	Docket Number (2) <div style="text-align: center;">0   5   0   0   0   4   5   4</div>	Page (3) <div style="text-align: center;">1   of   0   2</div>
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Title (4) REACTOR TRIP

Event Date (5)			LER Number (6)				Report Date (7)			Other Facilities Involved (8)		
Month	Day	Year	Year	Sequential Number	Revision Number		Month	Day	Year	Facility Names	Docket Number(s)	
0	4	0	2	8	5	8	5	0	3	9	0	0
										None	0   5   0   0   0	

OPERATING MODE (9) <div style="text-align: center;">1</div>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)									
POWER LEVEL (10) <div style="text-align: center;">0   3   0</div>	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input checked="" type="checkbox"/> X	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)					
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/>	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)					
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/>	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> Other (Specify in					
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/>	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	Abstract below and					
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/>	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	in Text)					
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/>	<input type="checkbox"/> 50.73(a)(2)(x)						

## LICENSEE CONTACT FOR THIS LER (12)

Name <div style="text-align: center;">Richard M. Williams, System Test Engineer      Ext. 385</div>	TELEPHONE NUMBER AREA CODE <div style="text-align: center;">8   1   5   2   3   4   -   5   4   4   1</div>
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## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	
D				N							

## SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> Yes (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	Expected Submission Date (15) <div style="text-align: center;">Month   Day   Year</div>
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## ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2:10 p.m. CST, with reactor power at 30%, a reactor trip occurred due to 1C Steam Generator LO-2 level. The low Steam Generator level was caused by the loss of feedwater flow from the 1B Feedwater Pump after it was placed from Manual to Automatic Control. It has been determined that the Byron General Procedure used for power ascension placed the Feedwater Pump Controller in automatic prematurely. The procedure has been revised.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)					Page (3)		
		Year	///	Sequential Number	///	Revision Number			
Byron, Unit 1	0   5   0   0   0   4   5   4	8   5	-	0   3   9	-	0   0	0   2	OF	0   2

TEXT

On April 2, 1985 at 2:10 P.M. CST, Unit 1 was at 30% power. The 1A Motor Driven Feedwater Pump and the Start-up Feedwater Pump were shutdown but available. The 1C Feedwater Pump was out of service. Feedwater to the Steam Generators was being delivered through the Feedwater Regulating Valve Bypass Regulating Valves and the Feedwater Upper Nozzle Tempering Flow line, the normal low power feedwater line-up.

Per Byron General Procedure 100-3 (power ascension), the Master Feedwater Pump Speed Controller was placed from Manual Control into Automatic Control. Immediately the 1B Feedwater Pump (Turbine Driven) speed and flow rate began to oscillate. The Unit Operator, noticing this transient, placed the Speed Controller back in Manual Control in an attempt to dampen the oscillations and bring the pump under control. These oscillations caused the High Pressure Steam Supply Stop Valve to close, eliminating the pumping capability of the pump.

The Unit Operator successfully started the 1A Motor Driven Feedwater Pump, but it tripped on low suction pressure. Another unsuccessful attempt was made to start this pump. The reactor then tripped due to 1C Steam Generator LO-2 level. No Safety Injection occurred and the plant was stabilized in Hot Standby.

It was determined that the 1A Motor Driven Feedwater Pump low suction pressure was due to a throttled Condensate Polishing System Supply Valve. It indicated open but was discovered to have a broken valve stem key and the valve was throttled 20% open. This starved the Feedpump suction. This valve has been repaired.

It was determined that the Master Feedwater Pump Speed Controller should not be placed in Automatic until the Main Feedwater Regulating Valves are in service and the Main Feedwater Isolation Valves are open. The Byron General Procedure has been revised accordingly.

This event, the loss of a Feedwater Pump, has occurred before at Byron (LER #85-031-00). A similar low FW pump suction pressure event has also previously occurred at Byron (LER #85-030-00). This event posed no threat to public safety. The licensed personnel on shift and the Reactor Protection Systems performed appropriately to bring this event to a safe conclusion. No further action is required.



**Commonwealth Edison**  
Byron Nuclear Station  
4450 North German Church Road  
Byron, Illinois 61010

April 22, 1985

LTR: BYRON 85-0604

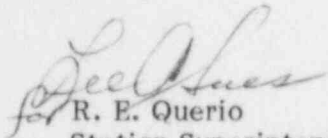
U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2)(iv) which requires a 30 day written report.

This report is number 85-039-00, Docket No. 50-454.

Very truly yours,



R. E. Querio  
Station Superintendent  
Byron Nuclear Power Station

REQ/gt

Enclosure: Licensee Event Report No. 85-039-00

cc: J. G. Keppler, NRC Region III Administrator  
J. Hinds, NRC Resident Inspector  
INPO Record Center  
CECO Distribution List

#3/017

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