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August 14, 1996  
6710-96-2274

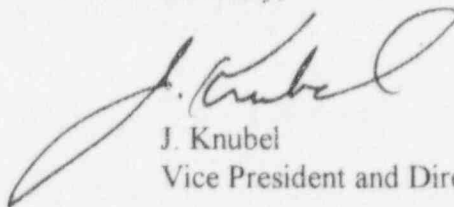
U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)  
Operating License No. DPR-50  
Docket No. 50-289  
Monthly Operating Report for July 1996

Enclosed are two copies of the July 1996 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,



J. Knubel  
Vice President and Director, TMI

WGH

9608220225 960731  
PDR ADOCK 05000289  
R PDR

cc: Administrator, Region I  
TMI Senior Resident Inspector  
96001

220038

IE2471

## OPERATIONS SUMMARY

July 1996

The plant entered the month operating at 100% power and remained at that power level for the remainder of the month. Net unit electrical output averaged approximately 796 MWe during July.

### MAJOR SAFETY RELATED MAINTENANCE

The major safety related maintenance items completed during the month involved the following equipment:

#### NR-V-13C Piping Repairs

A pin hole leak was found and repaired in the piping near NR-V-13C, a Nuclear Service Closed Cooler backwash valve. The repair involved welding a half coupling over the area of the leak and installing a pipe plug in the half coupling. The results of post-maintenance inspection were satisfactory and NR-V-13C and its associated piping were returned to service.

# OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE August 14, 1996  
 COMPLETED BY W G HEYSEK  
 TELEPHONE (717) 948-8191

## OPERATING STATUS

1. UNIT NAME: THREE MILE ISLAND UNIT 1  
 2. REPORTING PERIOD: JULY 1996  
 3. LICENSED THERMAL POWER: 2568  
 4. NAMEPLATE RATING (GROSS MWe): 872  
 5. DESIGN ELECTRICAL RATING (NET MWe): 819  
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe) 834  
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

### NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: \_\_\_\_\_

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): \_\_\_\_\_

10. REASONS FOR RESTRICTIONS, IF ANY: \_\_\_\_\_

### THIS MONTH YR-TO-DATE CUMMULATIVE

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	5111.0	192096.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	5111.0	114654.1
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	5111.0	113492.3
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1,907,510.4	13,067,216.6	279,182,142.6
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	626,781.0	4,363,570.0	93,817,044.1
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	592,582.0	4,123,392.0	88,166,796.1
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	59.1
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	59.1
21. UNIT CAPACITY FACTOR (USING MDC NET)		101.3	102.6	58.4
22. UNIT CAPACITY FACTOR (USING DER NET)		97.3	98.5	56.0
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	34.8
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289  
 UNIT TMI-1  
 DATE August 14, 1996  
 COMPLETED BY W G HEYSEK  
 TELEPHONE (717) 948-8191

MONTH: JULY

DAY AVERAGE DAILY POWER LEVEL  
 (MWe-NET)

1	791
2	795
3	802
4	804
5	803
6	799
7	794
8	789
9	791
10	802
11	804
12	802
13	794
14	789
15	788
16	791

DAY AVERAGE DAILY POWER LEVEL  
 (MWe-NET)

17	792
18	793
19	790
20	801
21	800
22	803
23	800
24	795
25	791
26	794
27	801
28	800
29	799
30	798
31	795

REPORT MONTH July 1996

DOCKET NO. 50-289  
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No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report#	System Code <sup>4</sup> & <sup>5</sup>	Component Code <sup>5</sup> & <sup>6</sup>	Cause & Corrective Action to Prevent Recurrence
						None			

<sup>1</sup>  
 F Forced  
 S Scheduled

<sup>2</sup>  
 Reason  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & Licensing Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method  
 1-Manual  
 2-Manual Scram  
 3-Automatic Scram  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for  
 preparation of Data Entry Sheets  
 for Licensee Event Report (LER)  
 File (NUREG-0161)

<sup>5</sup> Exhibit 1 same source

<sup>6</sup> Actually used exhibits F & I NUREG 0161

### REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 5, 1997**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **Yes. To support GPU Nuclear plans to do independent reload analyses for Cycle 12 as discussed in response to question 6 below, T.S. 6.9.5.2 would require revision to include references to the GPU Nuclear analysis methods applied to the reload.**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **A Technical Specification Change Request for the changes as discussed above will be submitted once the GPU Nuclear topicals are approved.**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
  - a) **GPU Nuclear Letter 6710-96-2092, dated March 28, 1996 confirmed plans to perform independent reload design evaluations for Cycle 12, the next operation cycle, based on NRC approved methods described in GPU Nuclear Topical Reports TR-091A (core physics), TR-087 (core thermal hydraulics), TR-078 (FSAR safety analyses) and TR-092P (design and setpoints methodology) submitted to the NRC. The latter three are in the NRC review and approval stage.**

**At this time, completion of the NRC review and issuance of NRC SERs is active. We have received questions on all reports and are in the process of providing answers. All remaining reports are expected to be approved in an acceptable time frame to support our reload design activities.**

**The GPU Nuclear Cycle 12 reload program and results are expected to be available for NRC review in the March to April 1997 time frame.**

b) Cycle 12 fuel rod performance calculations (e.g. internal pin pressure) will be performed by Framatome Cogema Fuels Company (FCF) using the approved TACO3 (BAW-10162P-A) and GDTACO (BAW-10184P-A) fuel codes. Results require minor changes to the Mark B9 fuel rod design (lower fill gas prepressure, increased plenum volume). The new design will meet all criteria in the latest approved revision of BAW-10179P-A, Safety Criteria and Methodology for Acceptable Cycle Reload Analyses. Fuel rod cladding corrosion calculations for all Cycle 12 fuel are being done by FCF using the COROS2 methodology now under review by the NRC with approval expected about September 1996. The TACO calculations are being done using power histories generated with the GPU Nuclear approved core physics codes CASMO-3/SIMULATE-3 (TR-091A). A letter requesting approval for use of the SIMULATE-3 power peaking uncertainty of 5.5% with the TACO methodologies, rather than the current FCF NEMO physics code (BAW-10180A, Rev 1) uncertainty of 4.8%, will be submitted in August 1996 with approval requested in September 1996.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 864
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained. Phase II is expected to be started in 2002.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through and beyond the end of the current operating license.