

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Joseph M. Farley - Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 6 4 1 OF 0 2	PAGE (3) 1 OF 0 2

TITLE (4) Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 3	3 0	8 5	8 5	0 0 9	0 0	0 4	2 9	8 5			0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
	20.402(b)	20.406(c)	X	50.73(a)(2)(iv)	73.71(b)					
POWER LEVEL (10) 0 19 1 5	20.406(a)(1)(i)	50.36(e)(1)		50.73(a)(2)(v)	73.71(c)					
	20.406(a)(1)(ii)	50.36(e)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
	20.406(a)(1)(iii)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(A)						
	20.406(a)(1)(iv)	50.73(a)(2)(iii)		50.73(a)(2)(viii)(B)						
	20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)						

LICENSEE CONTACT FOR THIS LER (12)									
NAME J. D. Woodard					TELEPHONE NUMBER AREA CODE 2 10 15 8 9 9 1 - 1 5 1 1 5 1 6				

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS
X	SIG I	ISV	H 2 1 3 1 0	N					

SUPPLEMENTAL REPORT EXPECTED (14)					EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO								

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2051 on 3-30-85, with the unit operating at 95% power, the reactor tripped due to low-low level in steam generator 2A. This was due to loss of the 2B steam generator feed pump (SGFP) on a low vacuum signal that occurred while condenser vacuum pressure transmitter PT502 was being calibrated. Plant operators took action to reduce reactor power to prevent a reactor trip following the SGFP trip, but were unsuccessful.

Following the reactor trip, the plant operators implemented FNP-2-EFP-0 (Reactor Trip or Safety Injection) and FNP-2-ESP-0.1 (Reactor Trip Response), ensuring the unit was safely in Mode 3. All safety systems functioned as designed.

This event was caused by leak-by of the isolation valve to PT502. This valve was replaced and the calibration of PT502 was completed. A caution statement concerning air inleakage will be added to the applicable procedures for condenser pressure transmitters and pressure switches. Health/safety of the public was not affected by this event.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Joseph M. Farley - Unit 2	0500036485	0	09	00	02	OF 02

TEXT (If more space is required, use additional NRC Form 365A's) (17)

At 2051 on 3-30-85, with the unit operating at 95% power, the reactor tripped due to low-low level in steam generator 2A. This was due to loss of the 2B steam generator feed pump (SGFP) on a low vacuum signal that occurred while condenser vacuum pressure transmitter PT502 was being calibrated. Plant operators took action to reduce reactor power to prevent a reactor trip following the SGFP trip, but were unsuccessful.

Following the reactor trip, the plant operators implemented FNP-2-EFP-0 (Reactor Trip or Safety Injection) and FNP-2-ESP-0.1 (Reactor Trip Response), ensuring the unit was safely in Mode 3. All safety systems functioned as designed. Operation of the steam dumps was blocked by the low condenser vacuum signal and all steam generator atmospheric relief valves opened. After the vacuum signal error was corrected, the steam dumps still would not operate due to a blown fuse. This fuse was replaced and the steam dumps were restored to operability.

During the calibration of PT502, isolation valve V538 was closed to isolate PT502 from pressure switches PS545, PS556 and the condenser. Due to leak-by of isolation valve V538, the pressure switches sensed decreasing vacuum when PT502 was opened to the atmosphere to apply a calibration signal. Low vacuum at PS556 caused the trip of the 2B SGFP and low vacuum at PS545 blocked operation of the steam dumps.

Isolation valve V538 was replaced. A caution statement concerning air inleakage will be added to the applicable procedures for condenser pressure transmitters and pressure switches.

Mailing Address

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600 North 18th Street
Post Office Box 2641
Birmingham, Alabama 35291
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R. P. McDonald
Senior Vice President
Flintridge Building



Alabama Power
the southern electric system

April 29, 1985

Docket No. 364

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Joseph M. Farley Nuclear Plant, Unit 2, Licensee Event Report No. LER 85-009-00 is forwarded in accordance with 10CFR50.73 to provide 30 day written notification of the occurrence.

If you have any questions, please advise.

Yours very truly,

R. P. McDonald

RPM/DSM:sam

Enclosure

xc: IE, Region II

IE22
1/1