

ATTACHMENT 2

PEACH BOTTOM ATOMIC POWER STATION
UNITS 2 AND 3

Docket Nos. 50-277
50-278

License Nos. DPR-44
DPR-56

TECHNICAL SPECIFICATION CHANGES

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1.0 DEFINITIONS (Cont'd)

Simulated Automatic Actuation - Simulated automatic actuation means applying a simulated signal to the sensor to actuate the circuit in question.

Site Boundary - That line beyond which the land is not owned, leased or otherwise controlled by licensee.

Source Check - A source check shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

Startup/Hot Standby Mode - In this mode the reactor protection scram trips, initiated by condenser low vacuum and main steam line isolation valve closure are bypassed, the reactor protection system is energized with IRM neutron monitoring system trip, the APRM 15% high flux trip, and control rod withdrawal interlocks in service. This is often referred to as just Startup Mode. This is intended to imply the Startup/Hot Standby position of the mode switch.

Surveillance Frequency - Periodic surveillance tests, checks, calibrations, and examinations shall be performed within the specified surveillance intervals. Specified periodic surveillance intervals are defined as:

(N) Hours	At least once per (N) hours
Shiftly	At least once per 12 hours
Daily	At least once per 24 hours
(N) Days	At least once per (N) days
Twice Per Week	At least once per 4 days
Weekly	At least once per 7 days
(N) Weeks	At least once per (7xN) days
Semi monthly	At least once per 15 days
Monthly	At least once per 31 days
2 Month	At least once per 61 days
Quarterly or 3 month	At least once per 92 days
Semi-annually or 6 month	At least once per 184 days
Annually or 12 month	At least once per 366 days
Once Per Cycle	At least once per 732 days
18 month	At least once per 550 days
Refuel	At least once per 732 days
(N) Years	At least once per (366xN) days
(N) Refuel Cycle	At least once per (732xN) days
24 Months	At least once per 732 days

These specified time intervals may be exceeded by 25%. Surveillance tests are not required on systems or parts of the systems that are not required to be operable or are tripped. If tests are missed on parts not required to be operable or are tripped, then they shall be performed prior to returning the system to an operable status.

A surveillance test of the diesel generators, that requires a plant outage, may be deferred beyond the calculated due date until the next refueling outage, provided the equipment has been similarly tested and meets the surveillance requirement for the other unit.

Transition Boiling - Transition boiling means the boiling regime between nucleate and film boiling. Transition boiling is the regime in which both nucleate and film boiling occur intermittently with neither type being completely stable.

Trip System - A trip system means an arrangement of instrument channel trip signals and auxiliary equipment required to initiate

TABLE 4.1.2

REACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENT CALIBRATION
MINIMUM CALIBRATION FREQUENCIES FOR REACTOR PROTECTION INSTRUMENT CHANNELS

Instrument Channel	Group (1)	Calibration (4)	Minimum Frequency (2)
IRM High Flux	C	Comparison to APRM on Controlled Shutdown	Maximum frequency once per week.
APRM High Flux			
Output Signal	B1	Heat Balance	Twice per week.
Flow Bias Signal	B1	With Standard Pressure Source	Every eighteen months.
LPRM Signal	B1	TIP System Traverse	Every 6 weeks.
High Reactor Pressure	B2	Standard Pressure Source	Once per operating cycle.
High Drywell Pressure	B2	Standard Pressure Source	Once per operating cycle.
Reactor Low Water Level	B2	Pressure Standard	Once per operating cycle.
High Water Level in Scram Discharge Instrument Volume	A	Water Column	Every refueling outage.
Turbine Condenser Low Vacuum	B2	Standard Vacuum Source	Once per operating cycle.
Main Steam Line Isolation Valve Closure	A	Note (5)	Note (5)
Main Steam Line High Radiation	B1	Standard Current Source (3)	Every 3 months.
Turbine First State Pressure Permissive	A	Standard Pressure Source	Every 6 months.

TABLE 4.2.B (CONTINUED)
MINIMUM TEST AND CALIBRATION FREQUENCY FOR CSCS

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
13) HPCI and RCIC Steam Line Low Pressure	(1)	Once/3 months	None
14) HPCI Suction Source Levels	(1)	Once/3 months	None
15) 4KV Emergency Power System Voltage Relays (HGA,SV)	Once/operating cycle	Once/5 years	None
16) ADS Relief Valves Bellows Pressure Switches	Once/operating cycle	Once/operating cycle	None
17) LPCI/Cross Connect Valve Position	Once/refueling cycle	N/A	N/A
18) Condensate Storage Tank Level (RCIC) (7)	Once/3 months	Once/operating cycle	Once/day
19) 4KV Emergency Power Source Degraded Voltage Relays (IAV,CV-6,ITE)	Once/month	Once/eighteen months	None

TABLE 4.2.F
MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

Instrument Channel	Calibration Frequency	Instrument Check
18) Drywell High Range Radiation Monitors	Once/operating cycle**	Once/month
19) Main Stack High Range Radiation Monitor	Once/eighteen months	Once/month
20) Reactor Bldg. Roof Vent High Range Radiation Monitor	Once/eighteen months	Once/month
21) Drywell Hydrogen Concentration Analyzer and Monitor	Quarterly***	Once/month

* Perform instrument functional check once per operating cycle.

** Channel calibration shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10R/hr and a one point calibration check of the detector below 10R/hr with an installed or portable gamma source.

*** At least a two-point calibration using sample gas.

PBAPS

3.6.D & 4.6.D BASESSafety and Relief Valves

The safety/relief and safety valves are required to be operable above the pressure (122 psig) at which the core spray system is not designed to deliver full flow. The pressure relief system for each unit at the Peach Bottom APS has been sized to meet two design bases. First, the total capacity of the safety/relief and the safety valves has been established to meet the overpressure protection criteria of the ASME code. Second, the distribution of this required capacity between safety/relief valves and safety valves has been set to meet design basis 4.4.4.1 of subsection 4.4 of the FSAR which states that the nuclear system safety/relief valves shall prevent opening of the safety valves during normal plant isolations and load rejections.

The details of the analysis which show compliance with the ASME code requirements is presented in subsection 4.4 of the FSAR and the Reactor Vessel Overpressure Protection Summary Technical Report presented in Appendix K of the FSAR.

Eleven safety/relief valves and two safety valves have been installed on Peach Bottom Unit 3 with a total capacity of 79.51% of rated steam flow. The analysis of the worst overpressure transient demonstrates margin to the code allowable overpressure limit of 1375 psig.

To meet the power generation design basis, the total pressure relief system capacity of 79.51% has been divided into 65.96% safety/relief (11 valves) and 13.55% safety (2 valves). The analysis of the plant isolation transient shows that the 11 safety/relief valves limit pressure at the safety valves below the setting of the safety valves. Therefore, the safety valves will not open.

Experience in safety/relief and safety valve operation shows that a testing of 50 per cent of the valves per cycle is adequate to detect failure or deteriorations. The safety/relief and safety valves are benchtested every second.

PBAPS

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.7.A Primary Containment (Cont'd.)3. Pressure Suppression Chamber-Reactor Building Vacuum Breakers

- a. Except as specified in 3.7.A.3.b below, two pressure suppression chamber-reactor building vacuum breakers shall be operable at all times when primary containment integrity is required. The setpoint of the differential pressure instrumentation which actuates the pressure suppression chamber-reactor building vacuum breakers shall be 0.5 ± 0.25 psid.
- b. From and after the date that one of the pressure suppression chamber-reactor building vacuum breakers is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such vacuum breaker is sooner made operable provided that the repair procedure does not violate primary containment integrity.

4. Drywell-Pressure Suppression Chamber Vacuum Breakers

- a. When primary containment is required, all drywell-suppression chamber vacuum breakers shall be operable and positioned in the fully closed position (except during testing) except as specified in 3.7.A.4.b and c below.
- b. Drywell-suppression chamber vacuum breaker(s) may be "not fully seated" as shown by position indication if testing confirms that the bypass area is less than or equivalent to a one-inch diameter hole. Testing shall be initiated within 8 hours of initial detection of a "not fully seated" position

4.7.A Primary Containment (Cont'd.)h. Drywell Surfaces

The interior surfaces of the drywell and torus shall be visually inspected each operating cycle for evidence of deterioration. In addition, the external surfaces of the torus below the water level shall be inspected on a routine basis for evidence of torus corrosion or leakage.

3. Pressure Suppression Chamber-Reactor Building Vacuum Breakers

- a. The pressure suppression chamber-reactor building vacuum breakers shall be checked for proper operation every refueling outage. Associated instrumentation including setpoint shall be checked for proper operation every eighteen months.

4. Drywell-Pressure Suppression Chamber Vacuum Breakers

- a. Each drywell-suppression chamber vacuum breaker shall be exercised through an opening-closing cycle once a month.
- b. When it is determined that a vacuum breaker is inoperable for opening at a time when operability is required, all other operable vacuum breakers shall be exercised immediately and every 15 days thereafter until the inoperable vacuum breaker has been returned to normal service.
- c. Once per operating cycle each vacuum breaker shall be visually inspected

PBAPS

3.7.A & 4.7.A BASES (Cont'd.)

The design basis loss-of-coolant accident was evaluated at the primary containment maximum allowable accident leak rate of 0.5%/day at 56 psig. Calculations made by the AEC staff with leak rate and a standby gas treatment system filter efficiency of 90% for halogens and assuming the fission product release fractions stated in TID 14844, show that the maximum total whole body passing cloud dose is about 1.0 REM and the maximum total thyroid dose is about 14 REM at 4500 meters from the stack over an exposure duration of two hours. The resultant doses that would occur for the duration of the accident at the low population zone distance of 7300 meters are about 2.5 REM total whole body and 105 REM total thyroid. Thus, the doses reported are the maximum that would be expected in the unlikely event of a design basis loss-of-coolant accident. These doses are also based on the assumption of no holdup in the secondary containment resulting in a direct release of fission products from the primary containment through the filters and stack to the environs. Therefore, the specified primary containment leak rate and filter efficiency are conservative and provide margin between expected off-site doses and 10 CFR 100 guidelines.

The water in the suppression chamber is used only for cooling in the event of an accident; i.e., it is not used for normal operation; therefore, a daily check of the temperature and volume is adequate to assure that adequate heat removal capability is present.

Drywell Interior

The interiors of the drywell and suppression chamber are painted to prevent rusting. The inspection of the paint during each major refueling outage, assures the paint is intact. Experience with this type of paint at fossil fueled generating stations indicates that the inspection interval is adequate.

Post LOCA Atmosphere Dilution

In order to ensure that the containment atmosphere remains inerted, i.e. the oxygen-hydrogen mixture below the flammable limit, the capability to inject nitrogen into the containment after a LOCA is provided. During the first year of operation the normal inerting nitrogen makeup system will be available for this purpose. After that time the specifically designed CAD system will serve as the post-LOCA Containment Atmosphere Dilution System. By maintaining a minimum of 2000 gallons of liquid N_2 in the storage tank it is assured that a seven-day supply of N_2 for post-LOCA containment inerting is available. Since the inerting makeup system is continually functioning, no

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

- and one main stack noble gas monitor shall be operable and set to alarm in accordance with the methodology and parameters in the ODCM. From and after the date that both reactor building exhaust vent monitors or both main stack noble gas monitors are made or found to be inoperable for any reason, effluent releases via their respective pathway may continue provided at least two independent grab samples are taken at least once per 8 hrs. and these samples are analyzed for gross activity within 24 hours, and at least two technically qualified members of the facility staff independently verify the release rate calculations.
- c. One reactor building exhaust vent iodine filter and one main stack iodine filter and one reactor building exhaust vent particulate filter and one main stack particulate filter with their respective flow rate monitors shall be operable. From and after the date that all iodine filters or all particulate filters for either the reactor building exhaust vent monitor or the main stack monitor are made or found to be inoperable for any reason, effluent releases via their respective pathway may

shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:

1. Instrument indicates measured levels above the alarm setpoint.
 2. Instrument indicates a downscale failure. Additionally, an instrument check shall be performed every day.
- 4b. The reactor building exhaust vent and the main stack flow rate monitors shall be calibrated every 12 months. Additionally, an instrument check shall be performed every day.
- 4c. The reactor building exhaust vent and the main stack iodine and particulate sample flow rate monitors shall be calibrated every 12 months. Additionally, an instrument check shall be performed every day for the reactor building exhaust vent sample flow rate monitors, and every week for the main stack sample flow rate monitor.
- 4d. The main stack sample flow line Hi/Lo pressure switches shall be functionally tested every 6 months and calibrated every 24 months.

Table 4.15*

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

		Instrument*	Instrument*	Instrument*
		Check	Functional Test	Calibration
<u>Instruments and Sensor Locations#</u>				
1.	Triaxial Time-History Accelerographs			
a.	Containment Foundation (torus compartment)	M	SA	R
b.	Refueling Floor	M	SA	R
c.	RCIC Pump (Rm #7)	M	SA	R
d.	"C" Diesel Generator	M	SA	R
2.	Triaxial Peak Accelerographs			
a.	Reactor Piping (Drywell)	NA	NA	R
b.	Refueling Floor	NA	NA	R
c.	"C" Diesel Generator	NA	NA	R
3.	Triaxial Response-Spectrum Recorders			
a.	Cable Spreading Rm	M	SA	R
* <u>Surveillance Frequencies</u>				
M: every month				
SA: every 6 months				
R: every 24 months				
** Effective upon completion of installation.				
# Seismic instrumentation located in Unit 2.				

1.0 DEFINITIONS (Cont'd)

Simulated Automatic Actuation - Simulated automatic actuation means applying a simulated signal to the sensor to actuate the circuit in question.

Site Boundary - That line beyond which the land is not owned, leased or otherwise controlled by licensee.

Source Check - A source check shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

Startup/Hot Standby Mode - In this mode the reactor protection scram trips, initiated by condenser low vacuum and main steam line isolation valve closure are bypassed, the reactor protection system is energized with IRM neutron monitoring system trip, the APRM 15% high flux trip, and control rod withdrawal interlocks in service. This is often referred to as just Startup Mode. This is intended to imply the Startup/Hot Standby position of the mode switch.

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(N) Weeks	At least once per (7xN) days
Semi monthly	At least once per 15 days
Monthly	At least once per 31 days
2 Month	At least once per 61 days
Quarterly or 3 month	At least once per 92 days
Semi-annually or 6 month	At least once per 184 days
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24 Months	At least once per 732 days

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TABLE 4.1.2

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Instrument Channel	Group (1)	Calibration (4)	Minimum Frequency (2)
IRM High Flux	C	Comparison to APRM on Controlled Shutdown	Maximum frequency once per week.
APRM High Flux Output Signal	B1	Heat Balance	Twice per week.
Flow Bias Signal	B1	With Standard Pressure Source	Every eighteen months.
LPRM Signal	B1	TIP System Traverse	Every 6 weeks.
High Reactor Pressure	B2	Standard Pressure Source	Once per operating cycle.
High Drywell Pressure	B2	Standard Pressure Source	Once per operating cycle.
Reactor Low Water Level	B2	Pressure Standard	Once per operating cycle.
High Water Level in Scram Discharge Instrument Volume	A	Water Column	Every refueling outage.
Turbine Condenser Low Vacuum	B2	Standard Vacuum Source	Once per operating cycle.
Main Steam Line Isolation Valve Closure	A	Note (5)	Note (5)
Main Steam Line High Radiation	B1	Standard Current Source (3)	Every 3 months.
Turbine First State Pressure Permissive	A	Standard Pressure Source	Every 6 months.

TABLE 4.2.B (CONTINUED)
MINIMUM TEST AND CALIBRATION FREQUENCY FOR CSCS

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
13) HPCI and RCIC Steam Line Low Pressure	(1)	Once/3 months	None
14) HPCI Suction Source Levels	(1)	Once/3 months	None
15) 4KV Emergency Power System Voltage Relays (HGA,SV)	Once/operating cycle	Once/5 years	None
16) ADS Relief Valves Bellows Pressure Switches	Once/operating cycle	Once/operating cycle	None
17) LPCI/Cross Connect Valve Position	Once/refueling cycle	N/A	N/A
18) Condensate Storage Tank Level (RCIC) (7)	Once/3 months	Once/operating cycle	Once/day
19) 4KV Emergency Power Source Degraded Voltage Relays (IAV,CV-6,ITE)	Once/month	Once/eighteen months	None

TABLE 4.2.F
MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

Instrument Channel	Calibration Frequency	Instrument Check
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21) Drywell Hydrogen Concentration Analyzer and Monitor	Quarterly***	Once/month

* Perform instrument functional check once per operating cycle.

** Channel calibration shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10R/hr and a one point calibration check of the detector below 10R/hr with an installed or portable gamma source.

*** At least a two-point calibration using sample gas.

PBAPS

3.6.D & 4.6.D BASESSafety and Relief Valves

The safety/relief and safety valves are required to be operable above the pressure (122 psig) at which the core spray system is not designed to deliver full flow. The pressure relief system for each unit at the Peach Bottom APS has been sized to meet two design bases. First, the total capacity of the safety relief and the safety valves has been established to meet the overpressure protection criteria of the ASME code. Second, the distribution of this required capacity between safety/relief valves and safety valves has been set to meet design basis 4.4.4.1 of subsection 4.4 of the FSAR which states that the nuclear system safety/relief valves shall prevent opening of the safety valves during normal plant isolations and load rejections.

The details of the analysis which show compliance with the ASME code requirements is presented in subsection 4.4 of the FSAR and the Reactor Vessel Overpressure Protection Summary Technical Report presented in Appendix K of the FSAR.

Eleven safety/relief valves and two safety valves have been installed on Peach Bottom Unit 3 with a total capacity of 79.51% of rated steam flow. The analysis of the worst overpressure transient demonstrates margin to the code allowable overpressure limit of 1375 psig.

To meet the power generation design basis, the total pressure relief system capacity of 79.51% has been divided into 65.96% safety/relief (11 valves) and 13.55% safety (2 valves). The analysis of the plant isolation transient shows that the 11 safety/relief valves limit pressure at the safety valves below the setting of the safety valves. Therefore, the safety valves will not open.

Experience in safety/relief and safety valve operation shows that a testing of 50 per cent of the valves per cycle is adequate to detect failure or deteriorations. The safety/relief and safety valves are benchtested every second.

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.7.A Primary Containment (Cont'd.)3. Pressure Suppression Chamber-Reactor Building Vacuum Breakers

- a. Except as specified in 3.7.A.3.b below, two pressure suppression chamber-reactor building vacuum breakers shall be operable at all times when primary containment integrity is required. The setpoint of the differential pressure instrumentation which actuates the pressure suppression chamber-reactor building vacuum breakers shall be 0.5 ± 0.25 psid.
- b. From and after the date that one of the pressure suppression chamber-reactor building vacuum breakers is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such vacuum breaker is sooner made operable provided that the repair procedure does not violate primary containment integrity.

4. Drywell-Pressure Suppression Chamber Vacuum Breakers

- a. When primary containment is required, all drywell-suppression chamber vacuum breakers shall be operable and positioned in the fully closed position (except during testing) except as specified in 3.7.A.4.b and c below.
- b. Drywell-suppression chamber vacuum breaker(s) may be "not fully seated" as shown by position indication if testing confirms that the bypass area is less than or equivalent to a one-inch diameter hole. Testing shall be initiated within 8 hours of initial detection of a "not fully seated" position

4.7.A Primary Containment (Cont'd.)h. Drywell Surfaces

The interior surfaces of the drywell and torus shall be visually inspected each operating cycle for evidence of deterioration. In addition, the external surfaces of the torus below the water level shall be inspected on a routine basis for evidence of torus corrosion or leakage.

3. Pressure Suppression Chamber-Reactor Building Vacuum Breakers

- a. The pressure suppression chamber-reactor building vacuum breakers shall be checked for proper operation every refueling outage. Associated instrumentation including setpoint shall be checked for proper operation every eighteen months.
4. Drywell-Pressure Suppression Chamber Vacuum Breakers
 - a. Each drywell-suppression chamber vacuum breaker shall be exercised through an opening-closing cycle once a month.
 - b. When it is determined that a vacuum breaker is inoperable for opening at a time when operability is required, all other operable vacuum breakers shall be exercised immediately and every 15 days thereafter until the inoperable vacuum breaker has been returned to normal service.
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PBAPS

3.7.A & 4.7.A BASES (Cont'd.)

The design basis loss-of-coolant accident was evaluated at the primary containment maximum allowable accident leak rate of 0.5%/day at 56 psig. Calculations made by the AEC staff with leak rate and a standby gas treatment system filter efficiency of 90% for halogens and assuming the fission product release fractions stated in TID 14844, show that the maximum total whole body passing cloud dose is about 1.0 REM and the maximum total thyroid dose is about 14 REM at 4500 meters from the stack over an exposure duration of two hours. The resultant doses that would occur for the duration of the accident at the low population zone distance of 7300 meters are about 2.5 REM total whole body and 105 REM total thyroid. Thus, the doses reported are the maximum that would be expected in the unlikely event of a design basis loss-of-coolant accident. These doses are also based on the assumption of no holdup in the secondary containment resulting in a direct release of fission products from the primary containment through the filters and stack to the environs. Therefore, the specified primary containment leak rate and filter efficiency are conservative and provide margin between expected off-site doses and 10 CFR 100 guidelines.

The water in the suppression chamber is used only for cooling in the event of an accident; i.e., it is not used for normal operation; therefore, a daily check of the temperature and volume is adequate to assure that adequate heat removal capability is present.

Drywell Interior

The interiors of the drywell and suppression chamber are painted to prevent rusting. The inspection of the paint during each major refueling outage, assures the paint is intact. Experience with this type of paint at fossil fueled generating stations indicates that the inspection interval is adequate.

Post LOCA Atmosphere Dilution

In order to ensure that the containment atmosphere remains inerted, i.e. the oxygen-hydrogen mixture below the flammable limit, the capability to inject nitrogen into the containment after a LOCA is provided. During the first year of operation the normal inerting nitrogen makeup system will be available for this purpose. After that time the specifically designed CAD system will serve as the post-LOCA Containment Atmosphere Dilution System. By maintaining a minimum of 2000 gallons of liquid N_2 in the storage tank it is assured that a seven-day supply of N_2 for post-LOCA containment inerting is available. Since the inerting makeup system is continually functioning, no

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

and one main stack noble gas monitor shall be operable and set to alarm in accordance with the methodology and parameters in the ODCM. From and after the date that both reactor building exhaust vent monitors or both main stack noble gas monitors are made or found to be inoperable for any reason, effluent releases via their respective pathway may continue provided at least two independent grab samples are taken at least once per 8 hrs. and these samples are analyzed for gross activity within 24 hours, and at least two technically qualified members of the facility staff independently verify the release rate calculations.

- c. One reactor building exhaust vent iodine filter and one main stack iodine filter and one reactor building exhaust vent particulate filter and one main stack particulate filter with their respective flow rate monitors shall be operable. From and after the date that all iodine filters or all particulate filters for either the reactor building exhaust vent monitor or the main stack monitor are made or found to be inoperable for any reason, effluent releases via their respective pathway may

shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exist:

1. Instrument indicates measured levels above the alarm setpoint.
2. Instrument indicates a downscale failure.

Additionally, an instrument check shall be performed every day.

- 4b. The reactor building exhaust vent and the main stack flow rate monitors shall be calibrated every 12 months. Additionally, an instrument check shall be performed every day.
- 4c. The reactor building exhaust vent and the main stack iodine and particulate sample flow rate monitors shall be calibrated every 12 months. Additionally, an instrument check shall be performed every day for the reactor building exhaust vent sample flow rate monitors, and every week for the main stack sample flow rate monitor.
- 4d. The main stack sample flow line Hi/Lo pressure switches shall be functionally tested every 6 months and calibrated every 24 months.

Table 4.15**SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instrument*</u> <u>Check</u>	<u>Instrument*</u>	
	<u>Functional</u> <u>Test</u>	<u>Instrument</u> <u>Calibration</u>

Instruments and Sensor Locations#

1.	Triaxial Time-History Accelerographs			
a.	Containment Foundation (torus compartment)	M	SA	R
b.	Refueling Floor	M	SA	R
c.	RCIC Pump (Rm #7)	M	SA	R
d.	"C" Diesel Generator	M	SA	R
2.	Triaxial Peak Accelerographs			
a.	Reactor Piping (Drywell)	NA	NA	R
b.	Refueling Floor	NA	NA	R
c.	"C" Diesel Generator	NA	NA	R
3.	Triaxial Response-Spectrum Recorders			
a.	Cable Spreading Rm	M	SA	R

* Surveillance Frequencies

M: every month
 SA: every 6 months
 R: every 24 months

** Effective upon completion of installation.
 # Seismic instrumentation located in Unit 2.