



Omaha Public Power District  
444 South 16th Street Mall  
Omaha NE 68102-2247

August 15, 1996  
LIC-96-0113

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Mail Station P1-137  
Washington, D.C. 20555

Reference: Docket No. 50-285

**SUBJECT: July 1996 Monthly Operating Report (MOR)**

Enclosed please find the July 1996 MOR for Fort Calhoun Station (FCS)  
Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

T. L. Patterson  
Division Manager  
Nuclear Operations

TLP/d11

Enclosures

c: Winston & Strawn  
L. J. Callan, NRC Regional Administrator, Region IV  
L. R. Wharton, NRC Project Manager  
W. C. Walker, NRC Senior Resident Inspector  
R. J. Simon, Westinghouse  
INPO Records Center

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OMAHA PUBLIC POWER DISTRICT  
Fort Calhoun Station Unit No. 1

July 1996  
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of July 1996, the Fort Calhoun Station (FCS) operated at a nominal 100% power. Normal plant maintenance, surveillance, equipment rotation activities and scheduled on-line modifications were performed during the month.

On July 17th at 0949 hours, a 1-hour non-emergency notification was made to the NRC pursuant to 10 CFR 50.72(b)(1)(v), *Lost Emergency Notification System (ENS)*, due to spurious intermittent ringing of the ENS phone. The operability of the ENS phone was not affected, thus, the notification was made using the ENS phone. AT&T is troubleshooting the problem.

Three additional incore nuclear detectors failed in July 1996, rendering sixteen of the twenty-eight detectors strings inoperable. All failures have occurred in detectors that were installed during the 1995 Refueling Outage. The failed detectors are scheduled for replacement during the 1996 Refueling Outage.

In order to reduce fission gas releases to Containment and minimize doses received during outage activities, the 42-day 1996 Refueling Outage which was scheduled to commence on September 21, 1996 is now scheduled to begin on October 5, 1996. Around September 20th, it is planned that reactor power will be slowly reduced to take advantage of radioactive decay and continue the cleanup of the Reactor Coolant System (RCS).

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of July, no power operated relief valve (PORV) or primary system safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

The RCS leak rate was relatively steady throughout the month. The trend of minimal RCS leakage following the March 1996 outage continued. The July daily leak rates were constant at slightly above 0.1 gpm. No degrading trends were noted this month.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION  
AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
None	

5. SIGNIFICANT SAFETY-RELATED MAINTENANCE FOR THE MONTH OF JULY 1996

- Performed electrical work in the electrical control panel, fixed lube oil leaks and replaced thermostatic elements in temperature control valves for Emergency Diesel Generator DG-1.
- Repaired a leaking weld on the Demineralized Water inlet to Charging Pump CH-1B Discharge Isolation Valve CH-364.
- Replaced the solenoids on the Raw Water inlet/outlet valves HCV-2882A/B and HCV-2883A/B for the Component Cooling Water (CCW) Heat Exchangers AC-1C and AC-1D, respectively.
- Made a setpoint change to the Containment Noble Gas Radiation Monitor Remote Ratemeter RM-051 per the Technical Data Book.
- Replaced breaker 1B3B-4 for CCW Pump Motor AC-3A-M
- Repaired Raw Water Strainer Motor RC-12A-M to correct oil seepage.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I  
OPERATING DATA REPORT

DOCKET NO. 50-285  
UNIT FORT CALHOUN STATION  
DATE AUGUST 05, 1996  
COMPLETED BY D. L. LIPPY  
TELEPHONE (402) 533-6843

OPERATING STATUS  
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1. Unit Name: FORT CALHOUN STATION  
2. Reporting Period: JULY 1996

NOTES

3. Licensed Thermal Power (MWt): 1500  
4. Nameplate Rating (Gross MWe): 502  
5. Design Elec. Rating (Net MWe): 478  
6. Max. Dep. Capacity (Gross MWe): 502  
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:  
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:  
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	5111.0	200305.0
12. Number of Hours Reactor was Critical	744.0	4538.8	158246.8
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	4497.9	156478.4
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1113012.0	6572933.0	209259241.3
17. Gross Elec. Energy Generated (MWH)..	365674.0	2210878.0	69144603.2
18. Net Elec. Energy Generated (MWH)....	348443.5	2108754.3	65966123.1
19. Unit Service Factor.....	100.0	88.0	78.1
20. Unit Availability Factor.....	100.0	88.0	78.1
21. Unit Capacity Factor (using MDC Net)	98.0	86.3	71.1
22. Unit Capacity Factor (using DER Net)	98.0	86.3	69.6
23. Unit Forced Outage Rate.....	.0	7.9	4.1

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):  
REFUELING OUTAGE SCHEDULED TO COMMENCE ON OCTOBER 5, 1996 WITH A PLANNED DURATION OF 42 DAYS.

25. If shut down at end of report period, estimated date of startup: \_\_\_\_\_

26. Units in test status (prior to comm. oper.):      Forecast      Achieved

INITIAL CRITICALITY  
INITIAL ELECTRICITY  
COMMERCIAL OPERATION

N/A

\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

ATTACHMENT II  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	AUGUST 05, 1996
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

MONTH JULY 1996

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1	463
2	463
3	466
4	468
5	470
6	470
7	468
8	469
9	469
10	471
11	472
12	472
13	473
14	472
15	471
16	470

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17	470
18	469
19	466
20	464
21	466
22	468
23	468
24	466
25	466
26	467
27	468
28	470
29	468
30	467
31	468

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285  
UNIT NAME Fort Calhoun St.  
DATE August 6, 1996  
COMPLETED BY D. L. Lippy  
TELEPHONE (402) 533-6843

REPORT MONTH July 1996

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
None									

1  
F: Forced  
S: Scheduled

2  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
H-Other (Explain)

3  
Method:  
1-Manual  
2-Manual Scram  
3-Automatic Scram  
4-Other (Explain)

4  
Exhibit F - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File (NUREG-0161)

5  
Exhibit H - Same Source



Attachment IV  
Refueling Information  
Fort Calhoun Station Unit No. 1

Report for the month ending: <u>July 31, 1996</u>	
1. Scheduled date for next refueling shutdown.	October 5, 1996
2. Scheduled date for restart following refueling.	November 15, 1996
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment?	Yes
a. If answer is yes, what, in general, will these be?	1) Enrichment limit of spent fuel racks increased to at least 4.5 w/o from 4.2 w/o. 2) Allow use of ZIRLO® fuel cladding.
b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload?	N/A
c. If no such review has taken place, when is it scheduled?	N/A
4. Scheduled date(s) for submitting proposed licensing action and support information.	1) Spent fuel rack enrichment limit change was NRC approved July 30, 1996. 2) ZIRLO® change was submitted July 15, 1996.
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	N/A
6. The number of fuel assemblies: a) in the core b) in the spent fuel pool c) spent fuel pool storage capacity	133 Assemblies 618 Assemblies 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	2007 Outage
Prepared by: <u>M. J. G. [Signature]</u> Date: <u>8/6/96</u>	