



**Entergy
Operations**

Entergy Operations, Inc.

Route 3, Box 1370

Russellville, AR 72801

Tel 501-964-3100

November 13, 1992

1CAN119203

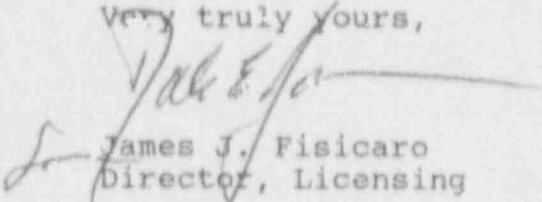
U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Stop P1-137
Washington, D.C. 20555

SUBJECT: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
Monthly Operating Report

Gentlemen:

Monthly Operating Report statistics for Arkansas Nuclear One, Unit 1, for October, 1992 is attached. This report is submitted in accordance with ANO-1 Technical Specification 6.12.2.3.

Very truly yours,


James J. Fisicaro
Director, Licensing

JJF/JRH/sjf
Attachment

cc: Mr. James L. Milhoan
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

NRC Senior Resident Inspector
Arkansas Nuclear One - ANO-1 & 2
Number 1, Nuclear Plant Road
Russellville, AR 72801

9211190269 921031
PDR ADOCK 05000313
R PDR

TE24

Mr. Thomas W. Alexion
NRR Project Manager, Region IV/ANO-1
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, Maryland 20852

Ms. Sheri Peterson
NRR Project Manager, Region IV/ANO-2
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, Maryland 20852

OPERATING DATA REPORT

DOCKET NO: 50-313
 DATE: November 6, 1992
 COMPLETED BY: K. R. Hayes
 TELEPHONE: (501) 964-5535

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 1
2. Reporting Period: October 1-31, 1992
3. Licensed Thermal Power (MWt): 2,568
4. Nameplate Rating (Gross MWe): 902.74
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____
9. Power Level To Which Restricted. If Any (Net MWe): None
10. Reasons For Restrictions. If Any: None

	<u>MONTH</u>	<u>YR-TO-DATE</u>	<u>CUMULATIVE</u>
11. Hours in Reporting Period	745.0	7320.0	156643.0
12. Number of Hours Reactor was Critical	745.0	5673.8	111535.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	5044.0
14. Hours Generator On-Line	745.0	5624.9	109357.7
15. Unit Reserve Shutdown Hours	0.0	0.0	817.5
16. Gross Thermal Energy Generated (MWH)	1912093	14197764	249370000
17. Gross Electrical Energy Generated (MWH)	650060	4809755	83000000
18. Net Electrical Energy Generated (MWH)	622194	4588857	78964105
19. Unit Service Factor	100.0	76.8	69.8
20. Unit Availability Factor	100.0	76.8	70.3
21. Unit Capacity Factor (Using MDC Net)	99.9	75.0	60.3
22. Unit Capacity Factor (Using DEC Net)	98.3	73.8	59.3
23. Unit Forced Outage Rate	0.0	0.1	11.9
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End of Report Period. Estimated Date of Start-up: _____

26. Units in Test Status (Prior to Commercial Operation): _____

	<u>Forecast</u>	<u>Achieved</u>
INITIAL CRITICALITY	_____	08/06/74
INITIAL ELECTRICITY	_____	08/17/74
COMMERCIAL OPERATION	_____	12/19/74

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-313
UNIT: One
DATE: November 6, 1992
COMPLETED BY: K. R. Hayes
TELEPHONE: (501) 964-5535

MONTH October, 1992

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	832
2	833
3	834
4	833
5	833
6	833
7	833
8	834
9	836
10	835
11	836
12	835
13	834
14	831
15	832
16	827
17	833
18	834
19	834
20	837
21	836
22	836
23	837
24	839
25	840
26	838
27	839
28	838
29	837
30	838
31	842

AVGS: 835

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Complete to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY

OCTOBER 1992

UNIT ONE

Unit one began the month operating at 100% power. On the sixteenth at 19:01 hours, the unit load was decreased to 92% to perform scheduled testing of the turbine throttle/governor valves. The unit power was then returned to 100% at 21:20 hours on the same day. The unit operated at full power for the remainder of the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS
REPORT FOR OCTOBER, 1992

DOCKET NO. 50-313
UNIT NAME ANO Unit 1
DATE November 6, 1992
COMPLETED BY K. R. Hayes
TELEPHONE 901-964-5535

<u>NO.</u>	<u>DATE</u>	<u>TYPE¹</u>	<u>DURATION (HOURS)</u>	<u>REASON²</u>	<u>METHOD OF SHUTTING DOWN REACTOR³</u>	<u>LICENSEE EVENT REPORT #</u>	<u>SYSTEM CODE⁴</u>	<u>COMPONENT CODE⁵</u>	<u>CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE</u>
------------	-------------	-------------------------	-----------------------------	---------------------------	--	--	------------------------------------	---------------------------------------	--

None

¹
F - Forced
S - Scheduled

²
Reason:
A - Equipment Failure (Explain)
B - Maintenance of Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & License Examination
F - Administration
G - Operational Error
H - Other (Explain)

³
Method:
1 - Manual
2 - Manual Scram.
3 - Automatic Scram.
4 - Continuation
5 - Load Reduction
9 - Other

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

⁵
Exhibit I - Same Source

DATE: October, 1992

REFUELING INFORMATION

1. Name of facility: Arkansas Nuclear One - Unit 1
2. Scheduled date for next refueling shutdown. September 17, 1993
3. Scheduled date for restart following refueling. November 12, 1993
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
Yes, Technical Specification change to increase fuel enrichment from 3.5% to 4.1%.
5. Scheduled date(s) for submitting proposed licensing action and supporting information. The Technical Specification change request was submitted to the NRC on June 27, 1991 (ICAN069108).
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
None.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 625
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
present 968 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
DATE: 1995 (loss of fullcore offload capability)