NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1): Limerick Generating Station - Unit 1

DOCKET NUMBER (2): 0 5 0 0 0 3 5 2

PAGE (3): 1 OF 0 3

TITLE (4): Engineered Safety Feature Actuation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 3	3 0	8 5	8 5	— 0 3 9	— 0 0	0 4	2 5	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
4	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
John C. Nagle, Engineer	AREA CODE 2 1 5 — 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)

	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)				
X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 85-039

On March 30, 1985, with Unit 1 in cold shutdown, an engineered safety feature actuation occurred. Test engineers performing an instrument valve "check-off list" initiated a spurious Nuclear Steam Supply Shutoff System isolation signal which closed the primary containment atmosphere sampling isolation valves (an ESF actuation). The instrument valve check-off list is performed periodically to verify the proper open or closed position of particular instrument rack valves. The isolation was immediately reset, and the sampling valves were re-opened. A formal procedure covering the instrument valve checkoff list is being written to mitigate similar occurrences. There were no adverse effects.

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Description of the Event:

On March 30, 1985 at 9:50 a.m. with Unit 1 in cold shutdown, test engineers performing an instrument valve "check-off list" initiated a spurious engineered safety feature actuation. In order to verify the "closed" postion of a valve on safeguard instrument rack 10C004, the engineers mistakenly opened the valve in question, causing a spurious Nuclear Steam Supply Shutoff System (NSSSS) isolation signal which closed the primary containment atmosphere sampling isolation valves SV57-133, SV57-183, and SV57-191, as designed.

Consequences of the Event:

At the time of the event, the unit was in cold shutdown. In this operating condition, primary containment sampling valve operability is not required. The isolation was reset and the valves re-opened within 5 minutes of the isolation. There were no adverse effects.

Cause of the Event:

NSSSS responds to various inputs such as low reactor water level, high drywell pressure, and high radiation. This ESF actuation was caused by a false low reactor water level signal caused by personnel error during the performance of an instrument valve check-off list not covered by an approved procedure.

Corrective Actions:

The isolation was reset and the valves re-opened within 5 minutes of the occurrence.

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Actions Taken To Prevent Reoccurence

Subsequent to this event, meetings involving the Instrument and Control technicians, test engineers, and Laboratory and Testing personnel were called to review the proper method of performing an instrument valve check-off list. Additionally, a procedure covering the method of performing an instrument valve check-off list is being generated to mitigate similar occurrences.

Previous Occurrences

None.

PHILADELPHIA ELECTRIC COMPANY

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PHILADELPHIA, PA. 19101

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April 25, 1985

Docket No. 50-352

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER deals with an inadvertent Engineered Safety Feature initiation caused by a valving error during the performance of an instrument valve check-off list.

Reference:	Docket No. 50-352
Report Number:	85-039
Revision Number:	00
Event Date:	March 30, 1985
Report Date:	April 25, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,

W. T. Ullrich
Superintendent
Nuclear Generation Division

TPM:vdw

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

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cc: Judge Helen F. Hoyt
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Atomic Safety & Licensing Board Panel
Docket & Service Section (3 Copies)
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January 16, 1985