

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Palo Verde Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 5 2 8				PAGE (3) 1 OF 2									
TITLE (4) Inadvertent Reactor Trip																							
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)										
0	3	2	1	8	5	8	5	0	0	9	0	0	0	4	1	9	8	5	0	5	0	0	0
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																					
5		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)									
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)									
0 0 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 365A)									
		20.405(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)													
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)													
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)													
LICENSEE CONTACT FOR THIS LER (12)																							
NAME W. F. Quinn (extension 4087)										TELEPHONE NUMBER 6 0 2 9 4 3 - 7 2 0 0													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS													
NA	NA	NA	NA	N																			
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 21, 1985 at 1320 an inadvertent reactor trip occurred during the performance of surveillance test 36ST-9SB02, Plant Protection System (PPS) Functional Test. The cause of the inadvertent trip was personnel error associated with a procedural deficiency. The PPS functioned as designed and opened the reactor trip breakers. No CEAs were withdrawn or being withdrawn at the time of the trip. A Procedure Change Notice (PCN) has been written and implemented in 36ST-9SB02 procedure to correct the deficiency.

8505030737 850419
PDR ADOCK 05000528
S PDRIE22
1/1

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Palo Verde Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 5 2 8	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	— 0 0 9	— 0 0	0	2 OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On March 21, 1985 at 1320 while in Mode 5 an inadvertent Reactor Trip occurred during the Performance of Surveillance Test 36ST-9SB02, Plant Protection System (PPS) Functional Test.

Initial performance of Surveillance Test procedure 36ST-9SB02 was in progress on channel "B" of the Plant Protection System (PPS). The technicians were beginning to perform section 8.5.5 which is a functional test of the low pressurizer pressure bypass verification.

The bistable select switch was positioned to parameter 25 (Low Pressurizer Pressure By-Pass) in accordance with the previous test, section 8.5.4. The bistable pushbutton was being depressed in accordance with 8.5.5.1. In step 8.5.5.3 the expected indication was not being observed on the panel meter.

In trying to discover why the panel meter indication was not changing, the bistable select switch was moved to parameter 6 (Low Pressurizer Pressure). After looking at Parameter 6 (Low Pressurizer Pressure), the bistable select switch was rotated back to parameter 25 (Low Pressurizer Pressure By-Pass). While the bistable select switch was being rotated, the bistable pushbutton was depressed. This resulted in a test voltage being applied to each parameter as the bistable select switch was rotated. For parameter 14 (Steam Generator 1 Low Flow), the test voltage resulted in a channel trip signal.

Concurrent with this action, a Steam Generator 1 Low Flow (Parameter 14) and Steam Generator 2 Low Flow (Parameter 15) condition existed in Channel "A". As the channel "B" bistable select switch was rotated through parameter 14 (Steam Generator 1 Low Flow), a 2 out of 4 trip logic condition existed. The PPS, as designed, initiated a reactor trip signal and all reactor trip breakers opened.

This inadvertent trip was caused by two factors. The first was a procedural deficiency to position the bistable select switch to parameter 6 (Low Pressurizer Pressure) at the beginning of section 8.5.5. A Procedure Change Notice (PCN) has been written and implemented in the procedure to correct the deficiency. The second was that the technicians should have ensured that the actions they were going to do to resolve the problems, with no increase on the panel meter, would not cause a reactor trip signal to be generated. A meeting was held with the technicians and both of these factors were fully discussed.



Arizona Nuclear Power Project

P.O. BOX 52034 • PHOENIX, ARIZONA 85072-2034

ANPP-32457-EEVB/WFQ

April 19, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Palo Verde Nuclear Generating Station (PVNGS)
Unit 1
Docket No. STN 50-528, License No. NPF-34
Licensee Event Report
File: 85-056-026; G.1.01.10

Dear Sirs:

Attached please find Licensee Event Report (LER) No. 85-009-00 prepared and submitted pursuant to 10 CFR 50.73. By copy of this letter we are also forwarding a copy of the LER to the Regional Administrator of the Region V Office.

If you have any questions or concerns, please contact me.

Very truly yours,

E. E. Van Brunt, Jr.
Executive Vice President
Project Director

EEVB/GEC/das
Attachment

cc: J.B. Martin
R.P. Zimmerman
A.L. Hon
E.A. Licitra
A.C. Gehr
INPO Records Center

LE22
11