



Wisconsin Electric POWER COMPANY

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April 26, 1985

CERTIFIED MAIL

Mr. H. R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. NUCLEAR REGULATORY COMMISSION
Washington, D. C. 20555

Attention: Mr. J. R. Miller, Chief
Operating Reactors, Branch 3

Gentlemen:

DOCKETS 50-266 AND 50-301
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 104
AUXILIARY BUILDING CRANE MODIFICATION
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

In accordance with Sections 50.59 and 50.90 of 10 CFR 50, Wisconsin Electric Power Company (Licensee) hereby requests amendments to Facility Operating Licenses DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units 1 and 2, respectively. The purpose of these amendments is to incorporate changes into the Point Beach Technical Specifications. These changes are necessitated by the pending modification of the auxiliary building rectilinear crane to a single-failure-proof status as requested by the NRC in NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants". Specific changes addressed by the application are discussed in the following. Proposed Technical Specification pages, with the changes identified by margin bars, are enclosed with this application.

A NRC letter from the Director, Division of Licensing, to all Licensees of Operating Plants dated December 22, 1980 directed implementation of NUREG-0612 by all licensees and applicants. This letter was supplemented by Generic Letter 81-07 dated February 3, 1981. In order to address the specific concern of handling heavy loads over or near the spent fuel pool using the overhead crane, we have contracted with Ederer, Incorporated to modify the auxiliary building crane in accordance with single-failure-proof criteria of Section 5.1.6 and Appendix C of NUREG-0612. This modification consists of replacement of the existing bridge trolley with an Ederer-designed trolley and associated hoists. This upgrade is scheduled to begin July 1, 1985, with tentative completion of installation and testing on August 1, 1985. This proposed revision to the Technical Specifications addresses changes in load limitations for crane movements over a spent fuel pool, the basis of those limitations, and surveillance of the crane and lifting devices.

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The proposed change to Technical Specification 15.3.8 includes deletion of Section B, "Limitations on Load Movements Over A Spent Fuel Pool". Those limitations were imposed as interim measures, restricting the loading and movements of a crane which did not meet single-failure-proof criteria. Also deleted is part of the basis which describes Section B.

Technical Specification 15.4.14, "Surveillance of Auxiliary Building Crane", would also be modified with this proposal. The surveillance involved inspection of limit switches which imposed restriction on crane bridge and trolley movements. The single-failure-proof modified crane would not be subject to such restrictions. Replacing this surveillance is a new surveillance requiring inspection of all slings and lifting devices prior to use in lifting heavy loads with the crane. Technical Specification 15.4.14 would be retitled "Surveillance of Auxiliary Building Crane Lifting Devices".

Ederer, Incorporated has published Revision 3 to Generic Licensing Topical Report EDR-1(P), "Ederer Nuclear Safety-Related Extra Safety and Monitoring (X-SAM) Cranes". The report describes the design and testing of the single-failure-proof features which are included in Ederer's X-SAM cranes intended for handling spent fuel casks and other safety-related loads in a nuclear plant. The report was accepted by the NRC in October 1982. The Appendix B Supplement and Appendix C Supplement to Topical Report EDR-1 are provided as enclosures to this letter. These supplements provide, respectively, a summary of plant-specific crane data supplied by Ederer, Incorporated and by the Licensee for the Point Beach Nuclear Plant auxiliary building crane. Additionally, the bridge crane seismic analysis specific to the Point Beach auxiliary building crane, as modified, is available upon request. Worthy of note is Page C-1 of the Appendix C Supplement, which states that a visual inspection of the existing crane bridge, including all accessible structural welds, will be conducted by a competent structural engineer. The Licensee contracted with Ederer, Incorporated to perform this inspection during the first week of April 1985. This inspection has been completed.

As required by 10 CFR 50.91(a)(1), we have prepared the following discussion concerning the issue of whether or not these proposed changes involve any significant hazards considerations. Such a determination must be made using the standards and criteria specified in 10 CFR 50.92. The Nuclear Regulatory Commission has provided guidelines concerning the application of these standards in 48 Federal Register 14870. Among the examples cited that are considered as very likely not to involve any significant hazards consideration is "a relief granted upon demonstration of acceptable operation from an operating restriction that was imposed because acceptable operation was not yet demonstrated". In the Licensee's submittal of March 21, 1978, as amended, limitations on loads and load paths for the auxiliary building crane were requested when it

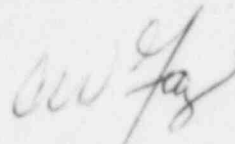
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was determined that installation of the single-failure-proof crane would be delayed several years. The NRC responded with the granting of License Amendment 35 to DPR-24 and License Amendment 41 to DPR-27 instituting those interim measures. Additionally, as requested by the NRC in the Staff's Technical Evaluation Report on heavy load handling, the Licensee's submittal of March 16, 1984, as amended, requested further, more restrictive, interim limitations on auxiliary building crane loading and movements. The NRC granted License Amendment 91 to DPR-24 and License Amendment 95 to DPR-27, effective April 28, 1985, instituting those restrictions. Our proposed revision to the Technical Specifications would delete these interim restrictions, their bases, and associated surveillance requirements based on the fact that the installation of the single-failure-proof crane trolley would make the current restrictions unnecessary to ensure the safe handling of heavy loads over or near the spent fuel pool. Accordingly, we believe that the proposed amendments to License Nos. DPR-24 and DPR-27 do not constitute a significant hazards consideration.

Enclosed are three signed originals and, under separate cover, forty copies of this amendments application. Also enclosed is a check in the amount of \$150 for the application fee specified in 10 CFR 170.12.

Please contact us if you have any questions concerning these proposed changes.

Very truly yours,



Vice President-Nuclear Power

C. W. Fay

Enclosures (Check No. 842047)

Copies to NRC Resident Inspector
R. S. Cullen, PSCW

Subscribed and sworn to before me
this 26th day of April 1985.


Notary Public, State of Wisconsin

My Commission expires May 4, 1986.