



Boston Edison

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier

Vice President Nuclear Operations
and Station Director

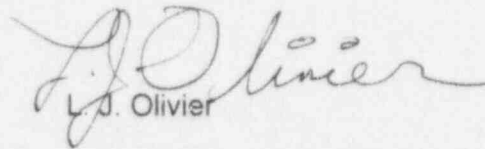
August 14, 1996
BECO Ltr. #96-074

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

JULY 1996 MONTHLY REPORT

In accordance with Pilgrim Nuclear Power Station Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report, please contact me directly.



L.J. Olivier

RLC/dmc/9458

Attachment

cc: Mr. Hubert Miller
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO.	50-293
NAME:	Pilgrim
COMPLETED BY:	R. L. Cannon
TELEPHONE:	(508) 830-8321
REPORT MONTH	July 1996

OPERATING STATUS

NOTES

- | | | |
|----|---|-----------|
| 1. | Unit Name | Pilgrim I |
| 2. | Reporting Period | July 1996 |
| 3. | Licensed Thermal Power (MWt) | 1998 |
| 4. | Nameplate Rating (Gross MWe) | 678 |
| 5. | Design Electrical Rating (Net MWe) | 655 |
| 6. | Maximum Dependable Capacity (Gross MWe) | 696 |
| 7. | Maximum Dependable Capacity (Net MWe) | 670 |
| 8. | If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons: | |

No Changes

- | | |
|-----|--|
| 9. | Power Level To Which Restricted, If Any (Net MWe): <u>None</u> |
| 10. | Reasons For Restrictions, If Any: <u>N/A</u> |

		<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11.	Hours in Reporting Period	744.0	5111.0	207263.0
12.	Hours Reactor Critical	744.0	4996.5	131262.6
13.	Hours Reactor Reserve Shutdown	0.0	0.0	0.0
14.	Hours Generator On-Line	744.0	4977.9	126801.8
15.	Hours Unit Reserve Shutdown	0.0	0.0	0.0
16.	Gross Thermal Energy Generated(MWH)	1455474.0	9704482.0	226006652.0
17.	Gross Electrical Energy Generated(MWH)	493700.0	3323460.0	76600784.0
18.	Net Electrical Energy Generated(MWH)	474818.0	3202360.0	73645982.0
19.	Unit Service Factor	100.0	97.4	61.2
20.	Unit Availability Factor	100.0	97.4	61.2
21.	Unit Capacity Factor (Using MDC Net)	95.3	93.5	53.0
22.	Unit Capacity Factor (Using DER Net)	97.4	95.7	54.2
23.	Unit Forced Outage Rate	0.0	0.0	11.6
24.	Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each) -	NONE PLANNED		
25.	If Shutdown at End of Report Period, Estimated Date of Startup -	UNIT OPERATING		

AVERAGE DAILY UNIT POWER LEVEL

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DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	650	17	662
2	652	18	661
3	649	19	661
4	649	20	661
5	648	21	660
6	649	22	660
7	646	23	660
8	647	24	658
9	647	25	659
10	645	26	657
11	645	27	658
12	597	28	657
13	231	29	658
14	626	30	656
15	662	31	657
16	657		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATIONAL SUMMARY

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REPORT MONTH	<u>July 1996</u>

The unit started the reporting period at 100% core thermal power (CTP) where it was maintained until 2022 hours on July 12, 1996. At that time, reactor power was reduced to approximately 50% CTP in support of a backwash of the main condenser. During the downpower it was identified that one of the reactor feed flow indicators was not tracking properly. The flow indicator was replaced and a reactor heat balance was satisfactorily performed. At 1924 hours on July 13, 1996, power ascension to 100% CTP commenced. At 0536 hours on July 14, 1996, 100% CTP was achieved and was maintained for the remainder of the reporting period.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF JULY 1996

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

DOCKET NO.	<u>50-293</u>
NAME:	<u>Pilgrim</u>
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REPORT MONTH	<u>July 1996</u>

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: February 1, 1997.
3. Scheduled date for restart following next refueling: March 14, 1997.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1765 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.
 - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

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REPORT MONTH July 1996

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER

No major safety related maintenance was completed during this reporting period.

UNIT SHUTDOWNS AND POWER REDUCTIONS

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NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
17	07/12/96	S	0.0	B	N/A	N/A	N/A	N/A	Planned down power to conduct main condenser back wash.

1	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)