

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Brunswick Steam Electric Plant Unit 1DOCKET NUMBER (2)
0 5 0 0 0 3 2 5 1 OF 0 1

TITLE (4)

Reactor Protection System Actuation During Unit 1 Refueling/Maintenance Outage

EVENT DATE (5)				LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)							
0	3	3	0	8	5	8	5	0	1	1	0	5	0	0	0		
											0	5	0	0	0		

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																														
5	<table border="1"><tr><td>20.402(b)</td><td>20.408(e)</td><td>X</td><td>80.73(a)(2)(iv)</td><td>73.71(b)</td></tr><tr><td>20.408(a)(1)(i)</td><td>80.38(a)(1)</td><td></td><td>80.73(a)(2)(v)</td><td>73.71(e)</td></tr><tr><td>20.408(a)(1)(ii)</td><td>80.38(a)(2)</td><td></td><td>80.73(a)(2)(vi)</td><td>OTHER (Specify in Abstract below and in Text, NRC Form 305A)</td></tr><tr><td>20.408(a)(1)(iii)</td><td>80.73(a)(2)(i)</td><td></td><td>80.73(a)(2)(viii)(A)</td><td></td></tr><tr><td>20.408(a)(1)(iv)</td><td>80.73(a)(2)(ii)</td><td></td><td>80.73(a)(2)(viii)(B)</td><td></td></tr><tr><td>20.408(a)(1)(v)</td><td>80.73(a)(2)(iii)</td><td></td><td>80.73(a)(2)(ix)</td><td></td></tr></table>	20.402(b)	20.408(e)	X	80.73(a)(2)(iv)	73.71(b)	20.408(a)(1)(i)	80.38(a)(1)		80.73(a)(2)(v)	73.71(e)	20.408(a)(1)(ii)	80.38(a)(2)		80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 305A)	20.408(a)(1)(iii)	80.73(a)(2)(i)		80.73(a)(2)(viii)(A)		20.408(a)(1)(iv)	80.73(a)(2)(ii)		80.73(a)(2)(viii)(B)		20.408(a)(1)(v)	80.73(a)(2)(iii)		80.73(a)(2)(ix)	
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20.408(a)(1)(ii)	80.38(a)(2)		80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 305A)																											
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20.408(a)(1)(v)	80.73(a)(2)(iii)		80.73(a)(2)(ix)																												

LICENSEE CONTACT FOR THIS LER (12)
NAME
M. J. Pastva, Jr., Regulatory Technician
TELEPHONE NUMBER
AREA CODE
9 1 1 9 4 5 7 - 2 3 1 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 3-30-85, at 1714, a full trip of both Unit 1 Reactor Protection System (RPS) Channels, A and B, and a Primary Containment Group I isolation occurred. The unit was in a refuel/maintenance outage. Removal of the detectors of the main steam line radiation monitors, 1-D12-RM-K603A, B, C, and D, General Electric Company Part No. 238X660G3, was in progress to facilitate access to the unit main steam isolation valve (MSIV) pit. An automatic trip of RPS Channel A circuitry occurred during work to remove the K603A detector. The K603A detector is in close proximity of the K603D detector. A noise spike was induced into the circuitry of the K603D detector during the work involving the K603A detector. The noise spike caused the K603D monitor to trip the RPS Channel B circuitry, thereby resulting in a simultaneous trip of both RPS channels. The incurred RPS trip and Group I were reset.

The automatic trips of the K603A and D monitors are attributed to sensitivity of the monitor circuitry to cross talk, signal noise, and spiking during work with, or in the immediate vicinity of, the monitor detectors or their signal cables.

By 11-5-85, an appropriate procedure will be developed and implemented to reduce the probability of future similar occurrences. Signs will be appropriately posted on both units to ensure personnel working in the vicinity of the detectors are aware of their sensitivity.

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Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
April 26, 1985

FILE: B09-13510C
SERIAL: BSEP/85-0757

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-85-011

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/jo

Enclosure

cc: Dr. J. N. Grace

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