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U.S. NUCLEAR REGULATORY COMMISSION  
Mail Station P1-137  
Washington, DC 20555

Gentlemen:

DOCKET 50-266  
REACTOR VESSEL OUTLET AND SI NOZZLE INDICATIONS  
POINT BEACH NUCLEAR PLANT, UNIT 1

In 1984, Wisconsin Electric Power Company (WE) contracted with Southwest Research Institute (SwRI) to examine portions of the Point Beach Nuclear Plant (PBNP) Unit 1 reactor vessel. During this examination, flaws in the outlet nozzle-to-shell welds were identified which were sized in excess of ASME Section XI allowable limits. These were addressed in Licensee Event Report (LER) 84-002 dated March 28, 1984. In accordance with IWB-2420(b) of ASME Section XI, 1977 Edition, Summer 1979 Addenda, the outlet nozzle-to-shell welds required successive examination during the next three inspection periods.

In 1987, WE again contracted with SwRI to examine the PBNP Unit 1 reactor vessel including a successive examination of the outage nozzle-to-shell welds as a result of the 1984 examination. During this vessel examination, flaws were identified in the SI nozzle-to-shell welds that exceeded ASME Section XI Code allowable limits, but were found acceptable for continued operation per evaluation in accordance with IWB-3600. This evaluation was submitted to you by our letter dated June 2, 1987. In accordance with IWB-2420(b) of ASME Section XI, 1977 Edition, Summer 1979 Addenda, the SI nozzle-to-shell welds required successive examination during the next three inspection periods.

In 1990, WE contracted with SwRI to perform the successive examinations of the outlet nozzle-to-shell and the SI nozzle-to-shell welds. These examinations were performed utilizing the SwRI Program and Remote (PaR) device. In addition, the SI nozzle-to-shell welds were examined with the SwRI Enhanced Data Acquisition System (EDAS) along with focused beam transducers to permit more

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Document Control Desk  
November 11, 1992  
Page 2

accurate sizing. This examination method description was submitted to you by our letter dated October 11, 1983. Based on the examination results using the focused beam technology, the flaws in the SI nozzle-to-shell welds were found to meet the ASME Section XI acceptance criteria. This evaluation was submitted to you by our letter dated April 30, 1990. Sizing of indications in the outlet nozzle-to-shell welds with focused beam transducers was not performed because of subsequent equipment problems. Conventional code techniques, which have a tendency to exaggerate the size of the indications were used instead.

Based on the results of the 1990 examinations, WE is planning to perform the final successive examination of the outlet nozzle-to-shell welds during the spring of 1993. This examination will be performed utilizing the SwRI PaR device, EDAS, and focused beam sizing transducers. The SI nozzle-to-shell welds are not subject to successive examination because the examination results in 1990 found the indications to be Code allowable. The SI nozzle-to-shell welds will be examined during the Reactor Vessel 10-year examination which is currently scheduled for the spring of 1997.

If you have any questions or require any additional information, please contact us.

Sincerely,



Bob Link  
Vice President  
Nuclear Power

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cc: NRC Regional Administrator, Region III  
NRC Resident Inspector