VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

August 9, 1996

Gentlemen: Director, Office Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

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VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION UNITS 1 AND 2
REVISED TESTING FREQUENCY OF THE POST
SIMULATOR CERTIFICATION FOUR-YEAR REPORTS

Pursuant to 10 CFR 55.45.(b)(5)(iii), we are submitting the Simulator Facility Certification Four Year Report for North Anna Units 1 and 2. The intent of these reports is to document our continuing compliance with ANSI/ANS-3.5-1985, as modified or endorsed by Regulatory Guide 1.149, dated April 1987. The reports are included as attachments to this letter.

If you have any questions, or require additional information please contact Dr. Arthur Friedman, Manager Nuclear Training at (804) 273-2701.

Very truly yours,

RF Saunders

R. F. Saunders
Vice President - Nuclear Operations

Attachments

A005

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Mr. R. D. McWhorter
NRC Senior Resident Inspector
North Anna Power Station

NORTH ANNA UNIT 1

SIMULATOR CERTIFICATION

2nd FOUR YEAR REPORT

(1992 - 1996)

NORTH ANNA UNIT 1

SIMULATOR CERTIFICATION SECOND FOUR YEAR REPORT

This North Anna Simulator Certification Four Year Report (1992 - 1996) consist of the following sections:

- Previous Four Year Simulator Test Results Summary (Attachment 1)
- 1997 - 2000 Simulator Test Schedule (Attachment 2)
- Simulator Fidelity & Upgrade Report (Attachment 3)
- Simulator Discrepancies Identified During NRC Examinations (Attachment 4)

ATTACHMENT 1

PREVIOUS FOUR YEAR SIMULATOR TEST RESULTS SUMMARY

Since August 1992, many modifications were made to the North Anna simulator. Testing, which verified simulator fidelity acceptable for training and certification, has been completed. Discrepancies uncovered during the previous four years are scheduled to be resolved in a timely manner. The testing schedule in the North Anna Unit 1 Simulator Certification 1992 four year report was satisfactorily completed, with no test failures.

The Real Time Test was conducted annually during the four year period. During 1994, 1995, and 1996 the test identified several quarter time peaks that exceeded 250 milliseconds. The duration of the peaks were less than one second in duration. Evaluation of the integrated response of the simulation showed that there were no discernible effects to the repeatability or functionality of the simulator. Various modifications have been carried out to resolve this matter.

The following malfunctions were modified to meet changing training requirements due to the installation of DCP 89-40-1, in 1995.

MRC15
MSS UNIT FAILURE HI/LOW

The Median Signal Selector unit failure malfunction was conducted initially on December 1, 1992 and again on April 21, 1995. the test was conducted at normal steady state full power conditions. The simulated microprocessor was tested in the failed high and failed low directions. Data was collected over the 30 second ramp time to demonstrate the degradation feature of the malfunction. The malfunction was degraded to its maximum severity. Various simulator model parameters points were monitored to verify the correct malfunction response. The duration of the test was 262 seconds. Data was collected in the form of computer printouts. There were no discrepancies noted during the test.

MRC17
RTD FAIL HI-LO HOT LEG

The Protection RTD Failure In Hot Leg malfunction was conducted initially on December 1, 1992 and again on April 21, 1995 the test was conducted at normal steady state full power conditions. The Channel I transmitter was tested in the failed high and failed low directions. Data was collected over the 30 second ramp time to demonstrate the degradation feature of the malfunction. The malfunction was degraded to its maximum severity. Various simulator model parameters points were monitored to verify the correct malfunction response. The duration of the test was 154 seconds. Data was collected in the form of computer printouts. There were no discrepancies noted during the test.

MRC18
RTD FAIL HI-LO COLD LEG

The Protection RTD Failure In Cold Leg malfunction was conducted initially on December 1, 1992 and again on April 21, 1995 the test was conducted at normal steady state full power conditions. The Channel I transmitter was tested in the failed high and failed low directions. Data was collected over the 30 second ramp time to demonstrate the degradation feature of the malfunction. The malfunction was degraded to its maximum severity. Various simulator model parameters points were monitored to verify the correct malfunction response. The duration of the test was 186 seconds. Data was collected in the form of computer printouts. There were no discrepancies noted during the test.

Based upon the testing conducted, and their results, the North Anna simulator is acceptable for licensed operator training and retraining.

ATTACHMENT 2

1997 - 2000 SIMULATOR TEST SCHEDULE

The next four year simulator test schedule follows. The tests are divided to ensure that 25% of the tests are performed each year thereby ensuring that all testing is completed within the four year time frame specified.

SIMULATOR PERFORMANCE TEST SCHEDULE

The performance testing conducted over the next four years will consist of the following tests.

- The Steady State Tests of ANS-3.5-1985 Appendix B section B.2.1 will be conducted annually.
- The Transient Performance Tests of ANS-3.5-1985 Appendix B section B.2.2 will be conducted annually.
- The Computer Real Time Test will be conducted annually.

August 1996 - July 1997

Integrated Tests

- Unit Startup from Cold Shutdown Condition (Mode 5) < 200°F with a solid pressurizer to Cold Shutdown Condition (Mode 5) at < 200°F.
- Unit Startup from Cold Shutdown Condition (Mode 5) to Hot Shutdown Condition (Mode 4) less than or equal to 350°F.

Malfunction Tests

MCA01	FAILURE OF CONTAINMENT INSTRUMENT AIR COMPRESSOR
MCA02	CONTAINMENT INSTRUMENT AIR LEAK
MCA04	INSTRUMENT AIR LEAK
MCC02	FAILURE OF COMPONENT COOLING PUMP TO START
MCC03	COMPONENT COOLING FLOW TRANSMITTER FAILURE
MCC04	COMPONENT COOLING PRESSURE TRANSMITTER FAILURE
MCC05	RCP THERMAL BARRIER LEAK
MCC06	COMPONENT COOLING PUMP SHEARED SHAFT
MCH01	ISOLABLE LETDOWN LINE LEAK IN CONTAINMENT
MCH02	ISOLABLE LETDOWN LINE LEAK OUTSIDE CONTAINMENT
MCH03	ISOLABLE CHARGING LINE LEAK OUTSIDE CONTAINMENT
MCH04	LETDOWN PRESSURE TRANSMITTER FAILURE

MCH05	REACTOR MAKEUP CONTROL BORATES IN THE AUTO MODE
MCH06	LOSS OF COMPONENT COOLING TO NRHX
MCH07	FAILURE OF CHARGING FLOW CONTROLLER
MCH08	FAILURE OF CHARGING FLOW TRANSMITTER
MCH09	REACTOR MAKEUP DILUTION IN AUTO MODE
MCH11	BORIC ACID LINE TO THE BLENDER PLUGGED
MCH12	VCT LEVEL TRANSMITTER FAILURE
MCH13	TUBE RUPTURE IN THE NON-REGENERATIVE HEAT EXCHANGER
MCH15	LOSS OF REACTOR MAKEUP CONTROL
MCH16	LOSS OF CHARGING PUMP
MCH18	LETDOWN PRESSURE CONTROL VALVE FAILS OPEN
MCH21	CHARGING PUMP CHECK VALVE STICKS OPEN
MCN01	LOSS OF CONDENSATE MAKEUP
MCN07	HIGH LEVEL DIVERT VALVE LCV-CN-107 FAILS OPEN
MCN14	CONDENSATE PUMP DISCHARGE CHECK VALVE STICKS OPEN
MCN16	CONDENSER AIR IN LEAKAGE
MCV01	GRADUAL INCREASE IN CONTAINMENT PRESSURE
MEL01	LOSS OF ALL OFF SITE POWER
MEL03	LOSS OF 416OV EMERGENCY BUS
MEL04	LOSS OF 416OV STATION BUS
MEL05	LOSS OF 125V D.C. BUS
MEL06	LOSS OF 480V EMERGENCY BUS
MEL07	LOSS OF 480V STATION BUS
MEL08	LOSS OF EMERGENCY DIESEL GENERATOR
MEL11	MAIN GENERATOR TRIP
MEL12	LOSS OF SEMI-VITAL BUS
MEL13	LOSS OF AC VITAL BUS INVERTER
MEL15	LOSS OF 480V EMERGENCY MCC
MEL16	LOSS OF 480V STATION SERVICE MCC
MEL17	LOSS OF PRIMARY PROCESS RACK POWER SUPPLY

August 1997 - July 1998

Integrated Tests

- Unit Startup from Hot Shutdown Condition (Mode 4) to Hot Standby Condition (Mode 3) at 547°F.

- Unit Startup from Hot Standby Condition (Mode 3) to Startup Condition (Mode 2) with Reactor critical at less than or equal to 5% power.

Malfunction Tests

MFW01	STEAM GENERATOR LEVEL TRANSMITTER FAILURE (0-100%)
MFW04	MAIN FEED PUMP LOW LUBE OIL PRESSURE
MFW05	MAIN FEEDWATER REGULATING VALVE FAILS CLOSED
MFW06	TOTAL LOSS OF FEEDWATER
MFW08	LOSS OF LEVEL ERROR SIGNAL TO S/G LEVEL CONTROL
MFW09	OVERSPEED TRIP OF AUXILIARY S/G FEED PUMP
MFW10	AUXILIARY FEED PUMP CHECK VALVE OPEN
MFW11	AUXILIARY FEED PUMP IMPELLER DEGRADATION
MFW12	MAIN FEED FLOW TRANSMITTER FAILURE
MFW13	MAIN FEEDWATER BREAK UPSTREAM OF FLOW TRANSMITTER
MFW14	AUX FEEDWATER BREAK DOWNSTREAM BETWEEN FLOW TRANS & CHECK VALVE
MFW15	MAIN FEEDWATER BREAK DOWNSTREAM OF CHECK VALVE OUTSIDE CONTAINMENT
MFW16	MAIN FEEDWATER BREAK IN CONTAINMENT
MFW17	DEGRADATION OF MAIN FEED PUMP
MFW18	MAIN FEEDWATER REGULATING VALVE FAILS OPEN
MFW19	MAIN FEEDWATER BREAK BETWEEN FLOW ELEMENT AND CHECK VALVE
MFW21	MAIN FEEDWATER PUMP SUCTION PIPE BREAK
MFW23	AUXILIARY FEEDWATER PUMP TRIPS ON OVERCURRENT
MGW01	ACCIDENTAL RELEASE OF RADIOACTIVE GAS
MMS01	STEAM FLOW TRANSMITTER FAILURE
MMS02	TURBINE FIRST STAGE PRESSURE TRANSMITTER FAILURE
MMS08	RUPTURE OF MAIN STEAM LINE IN CONTAINMENT BEFORE THE FLOW TRANSMITTER
MMS09	RUPTURE OF MAIN STEAM LINE AFTER THE NON-RETURN VALVE
MMS10	RUPTURE OF MAIN STEAM LINE BEFORE THE TRIP VALVE
MMS11	FAILURE OF STEAM DUMP CONTROL AS IS
MMS12	ATMOSPHERIC STEAM RELIEF VALVE STUCK OPEN
MMS13	MAIN STEAM TRIP VALVE FAILS AS IS
MMS14	MAIN STEAM SAFETY VALVE STUCK OPEN
MMS15	STEAM DUMP VALVE STUCK OPEN
MMS16	FAILURE OF STEAM HEADER PRESSURE CONTROL
MMS17	LOSS OF STEAM GENERATOR PRESSURE CHANNEL
MNI01	POWER RANGE UPPER DETECTOR FAILURE
MNI02	POWER RANGE LOWER DETECTOR FAILURE
MNI03	BLOWN POWER RANGE CONTROL FUSE
MNI04	BLOWN POWER RANGE INSTRUMENT FUSE
MNI06	SOURCE RANGE DETECTOR FAILURE (DISCRIMINATOR ERROR)
MNI08	INTERMEDIATE RANGE IMPROPERLY COMPENSATED
MNI09	INTERMEDIATE RANGE DETECTOR FAILURE
MNI10	SOURCE RANGE CHANNEL FAILURE
MNI13	POWER RANGE CHANNEL FAILURE

August 1998 - July 1999

Integrated Tests

- Unit Power Operation, Mode 2 to Mode 1(to include heat balance test).
- Unit Power Operation, Mode 1 to Mode 2.
- Computer Real Time Test.

Malfunction Tests

MQS01	LOSS OF QUENCH SPRAY PUMP
MQS03	FAILURE OF CONTAINMENT HIGH PRESSURE TO INITIATE SPRAY
MQS06	SPURIOUS CONTAINMENT SPRAY ACTUATION
MQS07	DEGRADATION OF INSIDE RECIRC SPRAY PUMP
MRC01	REACTOR COOLANT SYSTEM COLD LEG PIPE RUPTURE
MRC02	REACTOR COOLANT SYSTEM HOT LEG PIPE RUPTURE
MRC03	REACTOR COOLANT SYSTEM PUMP SUCTION LEG PIPE RUPTURE
MRC04	REACTOR COOLANT SYSTEM NON-ISOLABLE LEAK
MRC05	RCP OVERCURRENT TRIP
MRC06	FAILURE OF REACTOR VESSEL LEVEL TRANSMITTER
MRC07	PRESSURIZER PRESSURE TRANSMITTER FAILURE
MRC08	PRESSURIZER LEVEL TRANSMITTER FAILURE
MRC11	REACTOR COOLANT FLOW TRANSMITTER FAILURE
MRC14	FAILURE OF RCP NO. 3 SEAL
MRC15	MEDIAN SIGNAL SELECTOR (MSS) UNIT FAILURE HI/LOW
MRC17	RTD FAIL HI-LO HOT LEG
MRC18	RTD FAIL HI-LO COLD LEG
MRC19	PRESSURIZER RELIEF VALVE FAILS OPEN
MRC20	PRESSURIZER SPRAY VALVE STUCK OPEN
MRC21	PRESSURIZER SAFETY VALVE STUCK OPEN
MRC22	PRESSURIZER HEATERS FAIL ON (H BUS)
MRC23	J BUS PRESSURIZER HEATERS FAIL ON
MRC24	STEAM GENERATOR TUBE RUPTURE
MRC25	BOTH PRESSURIZER SPRAY VALVES FAIL CLOSED
MRC26	RCP SHEARED SHAFT
MRC29	PRESSURIZER PRESSURE CONTROL FAILS
MRC31	FUEL LEAK
MRC32	PCV-456 SEAT LEAKAGE
MRC33	PT-402 WIDE RANGE PRESSURE TRANSMITTER FAILURE
MRC34	PT-403 WIDE RANGE PRESSURE TRANSMITTER FAILURE
MRC35	PRT LEVEL TRANSMITTER FAILURE
MRC36	PRT PRESSURE TRANSMITTER FAILURE
MRC37	RCS VESSEL FLANGE LEAK TEMPERATURE DETECTOR FAILURE
MRC38	LOSS OF COMPONENT COOLING WATER TO RCP

MRC40	PZR SURGE LINE TEMPERATURE TRANSMITTER FAILURE
MRC41	RCS SPRAY LINE TEMPERATURE TRANSMITTER FAILURE
MRC42	PZR LIQUID/VAPOR TEMPERATURE TRANSMITTER FAILURE
MRC43	PRESSURIZER PORV TEMPERATURE TRANSMITTER FAILURE
MRC44	PZR SAFETY LINE TEMPERATURE TRANSMITTER FAILURE
MRC45	PRT LIQUID TEMPERATURE TRANSMITTER FAILURE

August 1999 - July 2000

Integrated Tests

- Unit Shutdown from Startup Condition with Power less than or equal to 5% (Mode 2) to Hot Standby Condition (Mode 3) at 547°F.
- Unit Shutdown from Hot standby Condition (Mode 3) to Hot Shutdown Condition (Mode 4) at less than 350°F.
- Unit Shutdown from Hot Shutdown Condition (Mode 4) to Cold Shutdown Condition (Mode 5) at less than or equal to 200°F.

Malfunction Tests

MRC46	PRESSURIZER SPRAY CONTROLLER FAILS
MRD01	ROD POSITION INDICATOR FAILURE
MRD06	UNCONTROLLED CONTINUOUS ROD WITHDRAWAL IN AUTO AT MAXIMUM SPEED
MRD07	UNCONTROLLED CONTINUOUS ROD INSERTION (AUTO) AT MAXIMUM SPEED
MRD08	CONTROL RODS MOVE AT MAXIMUM SPEED
MRD09	CONTROL RODS MOVE AT MINIMUM SPEED
MRD10	REVERSE DIRECTION IN AUTO ROD CONTROL
MRD11	RODS REVERSED DIRECTION IN MANUAL
MRD13	ROD CONTROL FAILURE CAUSING B & D BANKS TO MOVE TOGETHER
MRD14	RODS FAIL TO MOVE ON DEMAND IN AUTO
MRD15	RODS FAIL TO MOVE ON DEMAND IN MANUAL OR BANK SELECT
MRD16	DROPPED ROD
MRD21	EJECTED CONTROL ROD
MRD26	STUCK CONTROL ROD
MRD31	AUTO ROD CONTROL CONTROLS TEMPERATURE ABOVE SP
MRD32	FAILURE OF AUTO TRIP TO TRIP REACTOR ANY TIME
MRD33	FAILURE OF ALL ROD STOPS TO BLOCK ROD MOTION
MRD34	REACTOR TRIP BREAKERS OPEN
MRD36	LOSS OF NUCLEAR INSTRUMENT SIGNAL TO ROD CONTROL
MRD38	FAILURE OF REACTOR TRIP ON MANUAL DEMAND

MRH01	RESIDUAL HEAT REMOVAL SYSTEM LEAK
MRH02	RESIDUAL HEAT REMOVAL FLOW TRANSMITTER FAILURE
MRH03	LOSS OF COMPONENT COOLING TO RESIDUAL HEAT REMOVAL
MRH05	LOSS OF RESIDUAL HEAT REMOVAL PUMP
MRH06	RELIEF VALVE STUCK OPEN ON RESIDUAL HEAT REMOVAL SYSTEM
MRM01	AREA RADIATION MONITOR FAILURE
MRM02	PROCESS RADIATION MONITOR FAILURE
MRS01	LOSS OF OUTSIDE RECIRC SPRAY PUMP ON START
MRS02	LOSS OF INSIDE RECIRC SPRAY PUMP ON START
MRS04	RECIRC SPRAY NOZZLES CLOG
MRS05	TUBE LEAK IN RECIRC SPRAY HEAT EXCHANGER
MSI03	LOW HEAD SI PUMP SUCTION SCREEN CLOGS
MSI05	SPURIOUS SAFETY INJECTION
MSI06	FAILURE OF SAFETY INJECTION TO RESET
MSI07	FAILURE OF ONE SAFETY INJECTION TRAIN TO ACTUATE
MSI08	FAILURE OF ANY SAFETY INJECTION SIGNAL TO CAUSE SAFETY INJECTION
MSW01	SERVICE WATER PUMP TRIP
MTU01	TURBINE TRIP DUE TO OVERSPEED
MTU02	FAILURE OF MANUAL TURBINE TRIP TO FUNCTION ON DEMAND
MTU03	FAILURE OF AUTO TURBINE TRIP TO FUNCTION ON DEMAND
MTU12	FAILURE OF AUTO TURBINE RUNBACK
MTU13	SPURIOUS TURBINE RUNBACK
MTU14	REHEATER SAFETY VALVE STUCK OPEN

ATTACHMENT 3

SIMULATOR FIDELITY & UPGRADE REPORT

PHYSICAL FIDELITY

Physical fidelity is annually verified with an item by item comparison of the Simulator Control Room to a series of Unit 1 Plant Control Room photographs that are taken during January. Identified discrepancies are scheduled for resolution during the current year.

The report includes identification of all unresolved Simulator Control Panel discrepancies; indicating the work to be performed based on training impact, cost effectiveness, and other considerations as appropriate. Also included are discrepancies identified as requiring no action. Generic Control Room/Panel differences have been identified as necessary.

The simulator Physical Fidelity Report is not included with this report, however, it is available for examination.

CONTROL ROOM AND SIMULATOR PANEL COMPARISON UPDATE

A review was conducted of the original submittal report on the North Anna Control Room and Simulator Comparison of Panel Layout and Environment. The review was performed to update the changes made during the prior four years and validate the original differences noted.

Panels that remain non-simulated because of their relative minor training value or their inoperable status in the reference plant are:

STATION AND DIESEL BATTERY MONITORING PANEL
SEISMIC PANEL
SPILLWAY SUPERVISORY PANEL
ROBERT SHAW FIRE PROTECTION PANEL
VIBRATION/LOOSE PARTS MONITORING PANEL

CONTROL ROOM AND SIMULATOR ENVIRONMENTAL REVIEW UPDATE

Environmental differences identified in the initial simulator certification report and subsequently updated by the initial four year certification report have been reviewed and found to have no impact on training. Based upon the review results, the simulator is acceptable for operator training and retraining.

OTHER SIMULATOR UPGRADES INSTALLED

- The simulator interface hardware and software manufactured by Computrol, was replaced due to obsolescence and the inability to obtain adequate vender support for repairs. The new system is manufactured by VMIC and is classified as an intelligent I/O control system. A point to point verification was performed to verify that all data points were receiving correct data. Any discrepancies found were resolved prior to releasing the simulator for use in normal training activities.

- The Residual Heat Removal System model was upgraded to improve mid-loop operation response.

- Meteorological Panel recorders were replaced.

- In 1994, completed the initial software conversion from Encore/Gold to UNIX Fortran and initial integration testing.

- In 1995 the simulator computer hardware configuration was upgraded from a Gould 32/8780 to a SUN SPARC 10 workstation. The SUN workstation contains 256 MB of memory and four 90 MHz central processors. Since approximately 95% of the extant simulation software consisted of Fortran code, especially all process models, the simulator code was ported to the new computer and recompiled. Minor coding changes were made to resolve divide-by-zero, overflow, underflow, and other similar calculation errors, which were exposed during the testing phase. The result of these coding changes improved the quality of the code being used.

  The remaining software which was in assembly language, consisting primarily of device handlers was rewritten into either Fortran, C, or C++ as necessary. All handlers were extensively tested both stand-along and in conjunction with the full simulator software configuration to ensure response reliability.

  No new models were installed during the software conversion process. Extensive simulator testing was conducted to ensure model integrity and that all simulator features responded the same as, or better, than before the simulator computer replacement.

- Reactor core cycle upgrades 9, 10, and 11 were installed upon completion of the respective station refueling cycle.

- The simulator's audio and video recording capability was enhanced by the addition of upgraded microphones and cameras.

- Fast chart advance features were installed on all Hagan recorders in the simulator.

- Implemented capability to reflect reactor coolant system vacuum fill operations.

- Implemented Station Blackout Diesel via the Alternate AC diesel modification.

- Completed an Instructor Console upgrade.

- Completed a Steam Generator upgrade to reflect results of steam generator replacement project.

- Modified malfunctions MRC-15, MRC-17, and MRC-18 due to the installation of DCP 89-40-1 (REACTOR COOLANT SYSTEM RTD BYPASS MANIFOLD REMOVAL).

ATTACHMENT 4

SIMULATOR DISCREPANCIES IDENTIFIED DURING NRC EXAMINATIONS.

The following simulator discrepancies were identified during NRC inspections from October 1993 to February 1996. Simulator discrepancies identified during other simulator training activities are available upon request.

SMR No.	DESCRIPTION
9310130905	AFW flow from FW-P-2 is slightly below required amount with a single S/G steam supply. Completed on 10-25-93.
9310141615	Provide ability to perform 1-OP-7.3, section 5.6, Sluice Accumulators. Completed on 04-25-94.
9310141617	Addition of simulation capability for CH-292. Completed on 04-28-94.
9509221232	SI cold leg flow indication displays flow under LOCA conditions and no HHSI pumps running. Completed on 10-26-95.

NORTH ANNA UNIT 2

SIMULATOR CERTIFICATION

2nd FOUR YEAR REPORT

(1992 - 1996)

NORTH ANNA UNIT 2

SIMULATOR CERTIFICATION SECOND FOUR YEAR REPORT

The North Anna Power Station is a two unit station, operating from a common control room. The respective unit control panels are identical in their configuration and layout with respect to the operator. A few auxiliary systems panels present a mirror image layout to the operator in order to maintain an overall balanced appearance of the control room.

This North Anna Unit 2 Simulator Certification Four Year Report consists of the following sections:

- Simulator Fidelity & Upgrade Report (Attachment 1)

There were no significant differences identified during the previous four years. The North Anna Unit 1 Simulator meets the Unit 2 training needs.

ATTACHMENT 1

SIMULATOR FIDELITY & UPGRADE REPORT

PHYSICAL FIDELITY

A review was conducted of the original submittal report on the North Anna Unit 2 Control Room and Panel Comparison and Environment. The review was performed to update the changes made during the prior four years and validate the original differences noted. Any additional differences identified as a result of the Unit 1 and Unit 2 Control Room Panel Comparison will be implemented via the discrepancy reporting process.

UNIT 2 EQUIPMENT PANEL (SIMULATOR)

The Unit 2 Equipment Panel, located in the simulator control room, contains indications and controls for Unit 2 components of systems. This equipment is located in the main control room specifically on Unit 2 panels, and is simulated only to the extent to allow training to be conducted on systems that require actions to be taken at Unit 2 locations.

ENVIRONMENTAL REVIEW

Environmental differences and differences between the simulator and the Unit 1 Control Room are discussed within the Control Room/Simulator Panel and Environment Comparison Report of the North Anna Unit 1 Simulator Certification Second Four Year Report.