



Commonwealth Edison
LaSalle County Nuclear Station
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Marseilles, Illinois 61341
Telephone 815/357-6761

November 9, 1992

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for October 1992.

Very truly yours,


for G. J. Diederich
Station Manager
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

OCTOBER 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

TABLE OF CONTENTS
(UNIT 1)

I. INTRODUCTION

II. REPORT

- A. SUMMARY OF OPERATING EXPERIENCE
- B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT
- D. LICENSEE EVENT REPORTS
- E. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
- F. UNIQUE REPORTING REQUIREMENTS
 - 1. Main Steam Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor critical, Generator on-line at 800 Mwe. (Unit in coastdown).
	0050	Reduced power level to 750 Mwe due to system load.
	0420	Increased power level to 800 Mwe.
2	1000	Reducing power at 150 Mwe per hour for upcoming refuel outage.
	1200	Downshifted Reactor Recirculation pumps, power level at 370 Mwe.
	2000	Power level at 163 Mwe.
	2206	Power level at 80 Mwe.
3	0031	Generator offline, refueling outage (L1R05) began.
	0645	Reactor subcritical.
31	2400	Reactor subcritical, Generator offline, refuel outage (L1R05) in progress.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-010-00	10/09/92	During performance of a Division I Alternate Rod Insertion relay logic test, with the mode switch in refuel, an inadvertant scram was received on low charging water pressure.
92-011-00	10/15/92	Technical Specification requirements for the posting and control of a High-High radiation area were not maintained.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L16097	Switchgear 136X 1AP21E	X-Relay.	None.	Replace X-relay.
L16210	'A' Diesel Generator Soakback Pump AC	Transformer 74 relay.	Soakback pump will not continue to run.	Replace transformer and 74 relay.
L18121	Source Range Monitor 'B'	Pre-regulator.	Setpoints erratic excessive relay chatter.	Replace pre-regulator.
L18125	Average Power Range Monitor APRM 'B'	Power supply and fuse.	Erratic readings.	Replaced 20V power supply and fuse.
L18148	Intermediate Range Monitor IRM 'A'	Inoperable inhibit switch.	Monitor failed downscale.	Replaced inhibit switch.
L18266	Main Steam Line Rad Monitor ID18-K610D	FEM to ammeter card.	None.	Replace ammeter card.

(No SOR Failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE November 9, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD:	October 1992		
GROSS HOURS IN REPORTING PERIOD:	745		
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323		
MAX DEPENDABLE CAPACITY (MWe-Net):	1,036		
DESIGN ELECTRICAL RATING (MWe-Net):	1,078		
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):	N/A		
4. REASON FOR RESTRICTION (IF ANY):			
	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	54.2	6,568.3	53,924.8
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	48.5	6,529.3	52,907.6
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh))	94,066	19,903,179	155,060,158
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	30,784	6,702,577	51,792,349
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	21,520	6,469,278	49,664,423
12. REACTOR SERVICE FACTOR (%)	7.4	89.7	69.6
13. REACTOR AVAILABILITY FACTOR (%)	7.4	89.7	71.7
14. UNIT SERVICE FACTOR (%)	6.5	89.2	68.3
15. UNIT AVAILABILITY FACTOR (%)	6.5	89.2	68.3
16. UNIT CAPACITY FACTOR (USING MDC) (%)	2.8	85.3	61.9
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	2.7	82.0	59.5
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	1.4	6.9
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE November 9, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: October 1992

DAY	POWER	DAY	POWER
-----	-----	-----	-----
1	769	17	-12
2	455	18	-11
3	-11	19	-11
4	-12	20	-11
5	-12	21	-11
6	-12	22	-11
7	-12	23	-11
8	-12	24	-11
9	-11	25	-11
10	-11	26	-11
11	-12	27	-11
12	-11	28	-11
13	-11	29	-11
14	-11	30	-11
15	-11	31	-11
16	-12		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
4	921003	S	696.5	C	I	Refuel outage (L1R05)

SUMMARY OF OPERATION:

The unit entered a scheduled refueling outage on 10/03/92. Scheduled return to service is 01/08/93.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations
(None.)
2. ECCS System Outages
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual
(None.)
4. Major changes to Radioactive Waste Treatment Systems.
(None.)
5. Indications of Failed Fuel Elements.
(None.)

(Unit 1)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
0653	1E22-S001	General diesel generator maintenance.
1061	1E22-F031	Install flushing spool for refuel outage.
1063	1E22-S001	Diesel generator and manual injection stop valve relay logic testing.
1083 1108 1126	1E22-SYSTEM	Boundry out of service for high pressure core spray condensate suction piping modification.
1084	1E22-SYSTEM	High pressure core spray boundry out of service for LIR05.
1109	1E22-F019	Administrative control.
1110	1E22-F038	Administrative control.
1209	1E12-F331D 1E12-F331C	Check valve inspections.
1253	1E12-F068B	Valve repack.
1259 1260	1E22-S001	Battery replacement.
1446	1A RHR System	Operation of shutdown cooling mode.
1452	1B RHR System	Operation of shutdown cooling mode.
1453 1467 1489	1E12-F063C	Fill suppression pool.
1499	1E12-B001B	Administrative control.
1507	1E12-F087A	Administrative control.
1511	1E12-F068B	Administrative control.
1520	1E12-F092B 1E12-F090B 1E12-F092C	Administrative control.
1526	1E22-F003	Change limit switches, valve repack.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

OCTOBER 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS
(Unit 2)

I. INTRODUCTION

II. REPORT

- A. SUMMARY OF OPERATING EXPERIENCE
- B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT
- D. LICENSEE EVENT REPORTS
- E. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
- F. UNIQUE REPORTING REQUIREMENTS
 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1110 Mwe.
	0030	Reduced power level to 945 Mwe due to system load.
	1000	Increased power level to 1120 Mwe.
2	0300	Reduced power level to 1000 Mwe due to system load.
	0800	Increased power level to 1120 Mwe.
4	0230	Reduced power level to 1000 Mwe due to system load.
	1200	Increased power level to 1120 Mwe.
	2330	Reduced power level to 845 Mwe due to system load.
5	1000	Increased power level to 1120 Mwe.
6	0030	Reduced power level to 850 Mwe due to system load.
	0900	Increased power level to 1120 Mwe.
	2330	Reduced power level to 810 Mwe due to system load.
7	1200	Increased power level to 1125 Mwe.
9	0230	Reduced power level to 995 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
10	0300	Reduced power level to 990 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
11	0130	Reduced power level to 850 Mwe due to system load.
12	1800	Increased power level to 1130 Mwe.
13	0000	Reduced power level to 1000 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
14	0100	Reduced power level to 840 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
15	1700	Reduced power level to 880 Mwe, placed the 'B' Turbine Driven Reactor Feed Pump out of service for maintenance.
16	0600	Increased power level to 1026 Mwe.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2) (CONTINUED)

Day	Time	Event
	1700	Reduced power level to 900 Mwe for maintenance on the 'B' Turbine Driven Reactor Feed Pump.
17	0600	Increased power level to 1020 Mwe.
	1100	Reduced power level to 900 Mwe due to system load.
	2200	Increased power level to 1130 Mwe.
21	0300	Reduced power level to 1000 Mwe due to system load.
	1000	Increased power level to 1135 Mwe.
23	0130	Reduced power level to 990 Mwe due to system load.
	1130	Increased power level to 1130 Mwe.
24	0400	Reduced power level to 895 Mwe due to system load.
	1100	Increased power level to 1130 Mwe.
25	0130	Reduced power level to 990 Mwe due to system load.
	0430	Reduced power level to 890 Mwe due to system load.
	1100	Increased power level to 1130 Mwe.
	2230	Reduced power level to 1000 Mwe due to system load.
26	0130	Reduced power level to 850 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
27	0200	Reduced power level to 1060 Mwe due to system load.
	1000	Increased power level to 1130 Mwe.
30	0200	Reduced power level to 1000 Mwe due to system load.
	1100	Increased power level to 1130 Mwe.
31	0200	Reduced power level to 875 Mwe due to system load.
	2400	Reactor critical, Generator on-line at 1130 Mwe.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

(None.)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-014-00	10/28/92	Improper performance of a local leak rate test of a primary containment isolation valve (2G33-F040).
92-015-00	10/29/92	Isolation of the Reactor Water Cleanup system due to a high differential flow signal.

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
(See Table 2)
2. Average Daily Unit Power Level.
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.
(See Table 4)

C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L18379	Low Pressure Core Spray Min-Flow Static- O-Ring Switch (SOR)	SOR outside its reject limit.	Redundant systems available.	Install new switch.

(See attached SOR failure report.)

SOR dp SWITCH FAILURE DATA SHEET

Equipment Piece Number: 2E21-N004

Model Number: 103-AS-B202-NX-JJTTX6

Serial Number: 89-2-3877

Application: Minimum Flow Bypass Differential Pressure Switch

Date and Time of Discovery: 10/13/92 2100 hours

Reactor Mode: 1 (Run) Power Level: 100%

Calibration Tolerance: 23.6 - 22.4 "WC

Nominal Setpoint: 23.0 "WC

Action Limits: < 21.3 or > 24.7 "WC

Reject Limits: < 20.6 or > 25.4 "WC

Technical Specification

Limits: > 3.19 "WC

As Found Setpoint: 25.5 "WC

Date and Time of Return to Service: 10/15/92 1530 hours

Model Number of Replacement Switch: 103-AS-B202-NX-JJTTX6

Serial Number of Replacement Switch: 92-5-7832

DVR Number: 1-2-92-074

Cause: Switch was found out of calibration. Apparent cause is unknown,
a thorough investigation is in progress.

Corrective Action: The switch was replaced.

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE November 9, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD:	October 1992		
GROSS HOURS IN REPORTING PERIOD:	745		
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323		
MAX DEPENDABLE CAPACITY (MWe-Net):	1,036		
DESIGN ELECTRICAL RATING (MWe-Net):	1,078		
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):	N/A		
4. REASON FOR RESTRICTION (IF ANY):			
	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	745.0	4,641.1	49,576.9
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	745.0	4,454.8	48,659.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh))	2,362,560	13,343,630	145,526,029
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	800,492	4,505,298	48,395,048
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	776,892	4,317,398	46,448,902
12. REACTOR SERVICE FACTOR (%)	100.0	63.4	70.4
13. REACTOR AVAILABILITY FACTOR (%)	100.0	63.4	72.8
14. UNIT SERVICE FACTOR (%)	100.0	60.9	69.1
15. UNIT AVAILABILITY FACTOR (%)	100.0	60.9	69.1
16. UNIT CAPACITY FACTOR (USING MD) (%)	100.7	56.9	63.6
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	96.7	54.7	61.1
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	9.6	12.8
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:	N/A		

TABLE 3
E.2. AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE November 9, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: October 1992

DAY	POWER	DAY	POWER
1	1,033	17	980
2	1,062	18	1,096
3	1,073	19	1,098
4	1,027	20	1,098
5	1,008	21	1,082
6	1,006	22	1,098
7	1,004	23	1,064
8	1,092	24	1,058
9	1,074	25	1,079
10	1,071	26	1,043
11	833	27	1,084
12	1,010	28	1,097
13	1,059	29	1,068
14	1,041	30	1,066
15	1,018	31	1,028
16	923		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
8	921011	F	0.0	H	4	Power reduction due to system load.

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system load and maintenance activities.

F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations
(None.)
2. ECCS System Outages
(See Table 5.)
3. Changes to the Off-Site Dose Calculation Manual.
(None.)
4. Major changes to Radioactive Waste Treatment Systems.
(None.)
5. Indications of Failed Fuel Elements.
(None.)

(Unit 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
2133	2DG08CB	Control switch replacement.
2161	2E21-C001	Administrative control.
2168	2DG01K	Lubrication.
2179	2E22-S001	Lubrication and inspection.
2180		