ENCLOSURE 1

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1
October 1992
Monthly Operating Report

I. OPERATIONS SUMMARY

During October, Fort Calhoun Station (FCS) operated continuously at a nominal 100% power level. On October 15, 1992 the Army Corps of Engineers began river flow reductions for the winter season. By the end of the month, the Missouri River level was approximately 986' 6". To allow for intake grid cleaning, FCS removed the circulating water pumps from service one at a time. Several times during the month, the main turbine control valves exhibited slight drift (both open and closed). The drift was generally limited to about 1% valve position. This corresponds to about a 0.2°F change in cold leg temperature, and two (2) to three (3) MWt (a variation of 0.00133 to 0.002 percent reactor power). Troubleshooting by the Instrument and Control Technicians recorded various voltages to find a cause for the drift. At the end of October, the results had been inconclusive with no identified cause. Troubleshooting continued at the end of the month.

On October 25, 1992, the "A" channel of nuclear instrumentation was declared inoperable due to fluctuations in measurements. Per the Technical Specifications, FCS entered Limiting Condition of Operation (LCO) 2.15 (1) and appropriate trip units of the Reactor Protective System were placed in bypass within one hour. Instrumentation and Control Technicians conducted troubleshooting and adjusted voltage to the channel, with limited success. Cabling outside containment for the "A" safety channel was switched with the "A" control channel. This resulted in the problem being exhibited on the "A" control channel drawer in the control room. A containment entry was made to trace the problem on the containment side. Technicians repaired a bad connection found in containment but this did not correct the fluctuation problem. Subsequently a temporary modification was installed to exchange the "A" safety channel with the "A" control channel, and also to exchange the "D" safety channel with the "B" control channel to maintain a symmetric excore monitoring configuration. FCS personnel completed the change first with the "A" channel, followed by the "D" channel. Both safety channels were declared operable October 31, 1992, and the LCO was exited. While the final analyses are being completed concerning the Shape Annealing Factor and the effects of Rod Shadowing, Operations placed restrictions on rod motion.

The following NRC inspections were completed during October 1992:

IER No.	Title
92-20	Emergency Exercise Inspection
92-22	Monthly Resident Inspection
92-25	Check Valve Program Inspection
92-27	Maintenance Program Inspection
92-28	Security Systems Inspection

The following LERs were submitted during October 1992:

LER No.	LER Date	Description
92-019, Rev. 1	10/22/92	CEA Drop Due to Clutch Coil Failure
92-010 Rev. 2	10/30/92	Auxiliary Steam in Room 57 Outside Design Basis (HELR)

A. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

None.

B. RESULTS OF LEAK RATE TESTS

The plant operated throughout the month at a nominal 100 percent power. With no reactivity changes, the leak rate as measured by a daily surveillance test was consistent.

The total RCS leak rate has averaged slightly over 0.1 gpm. This is a reduction from 0.2 gpm at the end of September.

75% of the total leakage was "known leakage", indicated by an increase in Reactor Coolant Drain Tank level. The remainder was "unknown leakage". The "unknown leakage" was minimal throughout the month. Trending of the containment sump level this month indicates that there was negligible unknown leakage in the containment.

The RCS leakage rate is steady with no adverse trends noted.

C. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None.

D. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF OCTOBER 1992

Significant safety related work performed during October 1992 included:

- Replaced valve and stainless steel tubing on discharge sample valve AC-1121 for the fuel transfer canal drain pump (AC-13A).
- Replaced suction/discharge valves and seats. packing, guide bushings and front and top cap gaskets on charging pump CH-1C.
- Performed troubleshooting on the inlet valve YCV-1045 for the steam driven Auxiliary Feedwater Pump (FW-10) to investigate seat leakage. Adjusted the packing and rebuilt the actuator, solving the leakage problem.

OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE NOVEMBER 09 1992
COMPLETED BY G. R. CAVANAUGH
TELEPHONE (402) 636-2474

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: OCTOBER 1992

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
NA

9. Power Level to which restricted, if any (Net MWe): NA

10. Reasons for restrictions, if any:
NA

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period	745.0	7320.0	167450.0
12. Number of Hours Reactor was Critical	745.0	4327.6	129146.3
13. Reactor Reserve Shutdown Hours	.0	.0	1309.5
14. Hours Generator On-line	745.0	4222.1	127599.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1113359.0	5753565.9	167377291.6
17. Gross Elec. Energy Generated (MWH)	376408.0	1921567.0	55097693.2
18. Net Elec. Energy Generated (MWH)	359495.9	1825897.0	52559648.4
19. Unit Service Factor	100.0	57.7	76.2
20. Unit Availability Factor	100.0	57.7	76.2
21. Unit Capacity Factor (using MDC Net)	101.0	52.2	68.2
22. Unit Capacity Factor (using DER Net)	101.0	52.2	66.4
23. Unit Forced Outage Rate	.0	17.3	4.4

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
NONE.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.):

		Forecast	Achieved
INITIAL CRITICALITY	N/A	_____	_____
INITIAL ELECTRICITY		_____	_____
COMMERCIAL OPERATION		_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	NOVEMBER 09 1992
COMPLETED BY	G. R. CAVANAUGH
TELEPHONE	(402) 636-2474

MONTH OCTOBER 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	481	17	485
2	480	18	486
3	479	19	485
4	479	20	486
5	479	21	485
6	479	22	484
7	480	23	481
8	482	24	481
9	484	25	479
10	484	26	480
11	484	27	483
12	483	28	483
13	483	29	484
14	483	30	483
15	484	31	484
16	485		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending October 1992

1. Scheduled dated for next refueling shutdown. — September 1993

2 Scheduled date for restart following refueling. — November 1993

3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be? — Incorporate specific requirements resulting from reload safety analysis.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — N/A

   c. If no such review has taken place, when is it scheduled? — N/A

4. Scheduled date(s) for submitting proposed licensing action and support information. — June 1993

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — None Planned

6. The number of fuel assemblies:

a) in the core	133 Assemblies
b) in the spent fuel pool	529 Assemblies
c) spent fuel pool storage capacity	729 Assemblies
d) planned spent fuel pool storage capacity	Planned to be increased with high density spent fuel racks.

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1995*

* Capability of full core offload of 133 assemblies lost. Reracking to be performed between the 1993 and 1995 Refueling Outages.

Prepared by T. J. Hett — Date 11-13-92

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE November 9, 1992
COMPLETED BY G. R. Cavanaugh
TELEPHONE (402) 636-2474

REPORT MONTH October 1992

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
None	--	--	--	--	--	--	--	--	During October 1992, the plant operated at a nominal 100% power.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)