

# Duquesne Light Company

Beaver Valley Power Station  
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U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555-0001

**Subject: Beaver Valley Power Station, Unit No. 1  
Docket No. 50-334, License No. DPR-66  
10 CFR 50.61 RT-PTS Submittal**

The purpose of this submittal is to provide the NRC updated information relative to the current projected values of the pressurized thermal shock reference temperature (RT-PTS) for the Beaver Valley Unit No. 1 reactor pressure vessel limiting material (plate B6903-1). The pressurized thermal shock (PTS) rule requires that the PTS submittal be updated (10 CFR 50.61(b)(1)) and accepted by the NRC whenever there is information that indicates a significant change in projected RT-PTS values. Attached is WCAP-14543, "Evaluation of Pressurized Thermal Shock for the Beaver Valley Unit 1 Reactor Vessel" which documents the revised projected RT-PTS values for the Unit No. 1 reactor vessel.

The Duquesne Light Company (DLC) previously reported projected RT-PTS values on December 16, 1991, for Unit No. 1 and submitted a flux reduction program on March 16, 1992. These submittals were made in accordance with the regulatory requirements in 10 CFR 50.61. The NRC was advised at that time that plate B6903-1 was the limiting material and that the RT-PTS screening criteria would be exceeded prior to expiration of the Beaver Valley Unit No. 1 operating license. On February 6, 1992, and April 20, 1993, the NRC documented their review of the DLC submittals and on September 6, 1994, DLC provided additional information. The NRC safety evaluation (April 20, 1993) recommended the use of a 34°F margin term in determining RT-PTS and suggested the subject be revisited and the margin term be revised at an appropriate time in the future. This submittal is the result of a recent analysis using current methodology and an evaluation of industry and Unit 1 surveillance capsule data.

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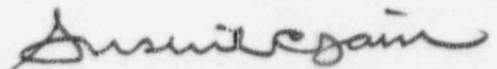
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Based on this analysis, the DLC has concluded that the RT-PTS values for the Unit No. 1 vessel beltline materials remain below the screening criteria of 10 CFR 50.61 using the projected end-of-license fluence for the limiting material. The attached WCAP provides the RT-PTS calculations for the Unit No. 1 beltline region materials for both current and end-of-license fluence values. This demonstrates Unit No. 1 will be able to operate for the entire duration of the operating license with margin to the RT-PTS screening criteria.

It is requested that the NRC complete the review and approval of this submittal and issue a revised safety evaluation by February 3, 1997. Questions regarding this submittal should be directed to Mr. Roy K. Brosi, Manager, Nuclear Safety at (412) 393-5210.

Sincerely,



Sushil C. Jain

c: Mr. L. W. Rossbach, Sr. Resident Inspector  
Mr. H. J. Miller, NRC Region I Administrator  
Mr. D. S. Brinkman, Sr. Project Manager