

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1): Joseph M. Farley - Unit 2										DOCKET NUMBER (2): 0 5 0 0 0 3 6 4				PAGE (3): 1 OF 01	
TITLE (4): Reactor Trip															
EVENT DATE (5):			LER NUMBER (6):				REPORT DATE (7):			OTHER FACILITIES INVOLVED (8):					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBERS:			
03	28	85	85	008	000	04	22	85				0 5 0 0 0			
OPERATING MODE (9):		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11):													
1		20.002(a)				20.006(a)				X 20.736(a)(2)(iv)				72.716(a)	
POWER LEVEL (10):		20.006(a)(1)(i)				20.006(a)(1)(ii)				20.736(a)(2)(v)				72.716(a)	
0.24		20.006(a)(1)(iii)				20.006(a)(1)(iv)				20.736(a)(2)(vi)				OTHER (Specify in Abstract, Draw and in Text, NRC Form 205A)	
		20.006(a)(1)(v)				20.726(a)(2)(i)				20.736(a)(2)(vii)(A)					
		20.006(a)(1)(vi)				20.726(a)(2)(ii)				20.736(a)(2)(vii)(B)					
		20.006(a)(1)(vii)				20.726(a)(2)(iii)				20.736(a)(2)(viii)					
		20.006(a)(1)(viii)				20.726(a)(2)(iv)				20.736(a)(2)(ix)					
LICENSEE CONTACT FOR THIS LER (12):															
NAME										TELEPHONE NUMBER					
J. D. Woodard										AREA CODE 205 899-5156					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15):		MONTH	DAY	YEAR	
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO					

ABSTRACT (Limit to 1600 spaces, i.e., approximately 8 lines single-space typewritten text) (16)

At 0427 on 3-28-85, the reactor tripped due to low-low level in steam generator 2A. Prior to the trip, reactor power had been reduced to 24% in preparation for the turbine overspeed test and the steam generator levels were being controlled manually using the main feed regulating valves and 2B steam generator feed pump (SGFP). A technician, while troubleshooting a problem with the 2A SGFP, was assigned to pull printed circuit cards C8-230 and C8-231 from 7300 process control cabinet number eight. However, he mistakenly pulled cards C8-231 and C8-232. Pulling card C8-232 caused the 2B SGFP to slow to approximately 3000 rpm, reducing steam generator feed flow and causing steam generator levels to decrease. Upon receipt of steam generator level deviation alarms, manual control of 2B SGFP was taken, but operators were unable to prevent the steam generator levels from decreasing to the low-low level reactor trip setpoint.

Following the trip, the operators implemented FNP-2-EEP-0 (Reactor Trip or Safety Injection) and FNP-2-ESP-0.1 (Reactor Trip Response), ensuring that the unit was safely in Mode 3. All safety systems functioned as designed.

This event was caused by personnel error. The technician involved has been counseled concerning his action. Health/safety of the public was not affected by this event.

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PDR ADOCK 05000364
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R. P. McDonald
Senior Vice President
Flintridge Building



April 22, 1985

Docket No. 364

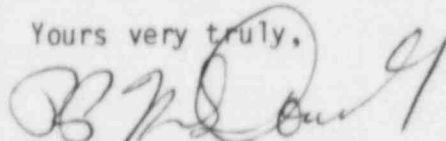
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U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Joseph M. Farley Nuclear Plant, Unit 2, Licensee Event Report No. LER 85-008-00 is forwarded in accordance with 10CFR50.73 to provide 30 day written notification of the occurrence.

If you have any questions, please advise.

Yours very truly,



R. P. McDonald

RPM/DSM:sam

Enclosure

xc: IE, Region II

IE22
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