

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Dresden Nuclear Power Station, Unit 2

DOCKET NUMBER (2)

0 5 0 0 0 2 3 7

PAGE (3)

1 OF 0 2

TITLE (4)

Main Steam Line Low Pressure Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)						
0	3	2	5	8	5	8	5	0	1	5	0	0	0	0	0	0
									N/A	0 5 0 0 0 0 0 0 0						
									N/A	0 5 0 0 0 0 0 0 0						

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following: (11))													
N		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)	
POWER LEVEL (10)		0 0 0				20.406(a)(1)(i)				50.36(e)(1)				73.71(c)	
		20.406(a)(1)(ii)				50.36(e)(2)				X 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)					
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(E)					
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)  
NAME  
Michael Moy (X-550)  
TELEPHONE NUMBER  
AREA CODE  
8 1 5 9 4 2 - 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	
X	J E	P S	B O 6 9	Y						

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While the unit was in an extended refuel outage, three main steam line low pressure switches tripped below the Technical Specification minimum trip point. The cause of the event was attributed to instrument setpoint drift. The pressure switches were immediately recalibrated to meet the Technical Specification limit and successfully tested several times. The safety significance was minimal since the setpoint at which the pressure switches tripped would still have prevented a major loss of coolant inventory.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Dresden Nuclear Power Station, Unit 2	0 5 0 0 0 2 3 7	8 5	0 1 5	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

While the unit was in an extended refuel outage, Main Steam Line Low Pressure Isolation Switch Calibration Test (DIS 250-2) was performed and pressure switches 2-261-30A, 2-261-30C and 2-261-30D tripped at 852 psig, 848 psig, and 845 psig respectively.

Compensating for a 5 psig head correction, the trip point should be greater than or equal to 855 psig per Technical Specification Table 3.2.1. The event was attributed to instrument setpoint drift. The MSL low pressure switches were immediately recalibrated to the greater than or equal to 855 psig setpoint and were tested several times. The pressure switches correctly tripped at the prescribed pressure. The function of the switches is to prevent a loss of inventory in the event of an EHC pressure regulator malfunction which would cause the turbine control valves and/or bypass valves to fully open. The Group I isolation would have occurred at 852 psig according to the four pressure switches' logic. Safety significance was minimal since this setpoint would still have prevented a major loss of coolant inventory if the unit was not shutdown in the event of an EHC pressure regulator failure. Last previous occurrence was reported by R.O. 85-01 on Docket #050-237.



**Commonwealth Edison**

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

April 22, 1985

DJS Ltr #85-431

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Licensee Event Report #85-015-0, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(vii).

D.S. Scott  
Station Manager  
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III  
File/NRC  
File/Numerical

LE22  
1/1