

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) LaSalle County Station Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 1 7 1 3										PAGE (3) 1 OF 0 3																									
TITLE (4) 3 SRV Open and Reactor Scram on Low Level																																													
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																											
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES																											
0 3		2 7		8 5		8 5		0 2		8 0		0 4		0 8		8 5		DOCKET NUMBER(S) 0 5 0 0 0 0 0 0 0 0																											
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																																											
3		20.402(b)										20.406(e)										X		80.73(a)(2)(iv)										73.71(b)											
POWER LEVEL (10)		0 0 0										20.408(a)(1)(i)										80.36(a)(1)												80.73(a)(2)(v)										73.71(a)	
		20.408(a)(1)(ii)										80.36(a)(2)												80.73(a)(2)(vi)																					
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LICENSEE CONTACT FOR THIS LER (12)																																													
NAME										TELEPHONE NUMBER																																			
John Ullrich, extension 571										AREA CODE 8 1 5 3 5 7 6 7 6 1																																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																													
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																																			
D	S B	Z Z Z Z	Z Z Z Z	N																																									
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)																																			
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO																																			
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																													
<p>On March 27, 1985, at 0408, a Unit 1 Reactor scram occurred due to low reactor vessel level. This low level was the result of the opening of safety relief valves during a leak test of instrument rack 1H22-P026.</p> <p>At the time of this event, Unit 1 was in Hot Shutdown, with a reactor pressure of 500 psig. When performing the leak test on instrument rack 1H22-P026, due to the discharge valve on the test pump being mispositioned, the test pressure jumped to 1120 psig in a few seconds. Pressure switches 1B21-N039B, J and S became pressurized because their root valves leaked causing safety relief valves 1B21-F013B, J and S to open.</p> <p>The event was of minimal significance because the plant responded as designed. No ECCS actuations occurred or were required. The plant was maintained in a safe condition at all times.</p> <p>The "B" Residual Heat Removal System was started in the Suppression Pool Cooling Mode to maintain Suppression Pool temperature. The reactor water level was slowly returned to normal using the Control Rod Drive System to minimize the amount of vessel cooldown.</p>																																													

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
LaSalle County Station Unit 1	0 5 0 0 0 3 7 3 8 5	—	0 2 8	—	0 0	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 388A's) (17)

I. EVENT DESCRIPTION

On March 27, 1985, at 0408, a Unit 1 Reactor scram (JM) occurred due to low reactor vessel level. This low level was the result of the opening of safety relief valves due to pressure switches 1B21-F013B, J and S being pressurized during a leak test of instrument rack 1H22-P026 sensing lines.

II. CAUSE

At the time of this event, Unit 1 was in Hot Shutdown, with a reactor pressure of 500 psig. Modification 1-1-84-091, which installs environmentally qualified switches on instrument rack 1H22-P026, had been performed to the point where a leak test of the affected instrument lines was required. A test pump was connected to the five valve manifold for instrument 1B21-N037AA, in order to pressurize the sensing lines of "D" Reactor Level and Pressure Instrument Rack, 1H22-P026 (AD, NB). The instrument root valves for the remaining instruments on this rack were closed and out of service. The pressure was increased slowly to 200 psig and held to notify the QC Inspector, then the increase to the test pressure (1046 psig) was started. Due to having the pressure control valve on the discharge of the test pump mispositioned, the test pressure jumped to 1120 psig within a few seconds. (This exceeded the 1.06 times the test pressure upper limit for the pressure test required by the Modification.) At this point Safety Relief Valves (SB, NB) 1B21-F013B, J and S opened for 15, 24 and 28 seconds respectively. This relief valve lift was unknown to the test pump operators. The test pump was stopped, the instrument lines were vented off and after closing down on the pump discharge valve, another attempt was made at performing the pressure test. When the pressure was increased to approximately 1050 psig, safety relief valve 1B21-F013S opened 2 more times (once for 37 seconds and then again for 1 minute 41 seconds). These lifts were again unknown to the test pump operators. During this second lift the Unit 2 Operating Shift Foreman entered the area where the test pump was located and told the crew to suspend testing. At about the same time a reactor scram (JC, RP) occurred, due to low vessel level (approximately 8 inches by Narrow Range indication) as a result of water inventory loss during the safety relief valve lifts. It appears that the instrument root valves for pressure switches 1B21-N039B, J and S leaked by thereby allowing the pressure switches to pressurize to the actuation setpoint.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

This event was of minimal significance. Excessive loss of reactor coolant did not occur. No ECCS actuations occurred or were required. The design pressure of the pressurized piping was not exceeded (design pressure 1250 psig).

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TEXT (If more space is required, use additional NRC Form 386A's) (17)

IV. CORRECTIVE ACTIONS

The "B" Residual Heat Removal System (BO, RH) was started in the Suppression Pool Cooling Mode to maintain the Suppression Pool water temperature within limits. The reactor water level was slowly returned to normal using the Control Rod Drive System (AA, RD) to minimize the amount of vessel cooldown. The instrument root valves for pressure switches 1B21-N039B, J and S, were tightened and the leak test was performed satisfactorily. Since safety relief valve "S" opened when only 1050 psig was applied to the instrument rack, Work Request L48283 has been written to verify the calibration of pressure switch 1B21-N039S. The required setpoint of this switch is 1076 psig \pm 12 psig. This pressure switch will be replaced as part of Modification 1-1-84-72. In addition, the following Action Item Records have been written: 1) 01-85-67048 has been written to track the review of overpressure protection requirements, and draw up guidelines to define when relief valves are required and what the respective setpoint should be during the performance of a hydrostatic or leak test. 2) 01-85-67049 is to track the completion of training of personnel who perform leak tests and hydrostatic tests, to caution them that process lines with small volumes take very little time to pressurize and that linear control of the test pump is required.

V. PREVIOUS OCCURRENCES

None.

VI. NAME AND TELEPHONE NUMBER OF PREPARER

John Ullrich, 815/357-6761, extension 571.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
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Telephone 815/357-6761

April 8, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #85-028-00, Docket #050-373 is being submitted to your office in accordance with 10CFR 50.73.

for R.D. Buehler
G. J. Diederich
Station Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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