



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401-1927
Telephone (612) 330-5500

November 6, 1992

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

10 CFR Part 50
Section 50.90

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Supplement 2 to License Amendment Request Dated March 20, 1992
Auxiliary Electrical System Changes
and
Cooling Water System Changes

On March 20, 1992, we submitted a License Amendment Request. This License Amendment Request reflects plant modifications being performed as part of NSP's Station Blackout/Electrical Safeguards Upgrade Project for Prairie Island Nuclear Generating Plant, Units 1 and 2.

As a result of two factors, we need to change the degraded voltage setpoints from those proposed in the original License Amendment Request. One factor is that performance problems with the new 480V voltage regulators is forcing us to delay placement of the regulators into service until later. The regulators will be placed into service during the subsequent refueling outage for each of the Units. Additionally, completion of a new voltage analysis for the plant reveals that degraded voltage setpoints must be increased from the existing settings until the new voltage regulators are placed into service. The new voltage analysis requires verification which has not yet been completed. We will confirm the validity of the proposed setpoints by letter by December 1, 1992. Both the design of the undervoltage protection scheme and the voltage analysis are in accordance with the guidance of Branch Technical Position PSB-1. The setpoint change is reflected in the attached revised page changes (TABLE TS.3.5-1 and R.3.5-4) to the Technical Specifications, Appendix A of the Operating Licenses, for Prairie Island Nuclear Generating Plant, Units 1 and 2. A separate License Amendment Request will be submitted to address the setpoint changes necessary when the new regulators are installed.

The discussion in the significant hazards analysis and safety evaluation submitted with the March 20, 1992 License Amendment Request still apply with the substitution of the new setpoint for the original setpoints and the understanding that the voltage regulators will not be made operational without a separate License Amendment Request.

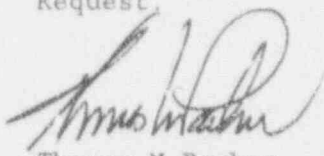
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In addition, our original submittal contained an error regarding surveillance testing requirements. TS.4.6.A.3.b.4 requires, during the Integrated Safety Injection test, the following: "Verify that the diesel generator system trips, except those for engine cv speed, ground fault, and generator differential current, are automatically bypassed." We had proposed no change to this specification. However, for Unit 2, the ground fault trip is automatically bypassed when a safety injection signal exists. Therefore, this difference is reflected in the attached revised page change (TS.4.6.2) to the Technical Specifications, Appendix A of the Operating Licenses, for Prairie Island Nuclear Generating Plant, Units 1 and 2. The significant hazards analysis and safety evaluation submitted with the March 20, 1992 License Amendment Request still apply.

Please contact us if you have any questions concerning this License Amendment Request.



Thomas M Parker
Manager
Nuclear Support Services

c: Regional Administrator-Region-III, NRC
NRR Project Manager, NRC
Senior Resident Inspector, NRC
State of Minnesota
Attn: Kris Sanda
J E Silberg

Attachments:

Affidavit
Exhibit A - Revised Technical Specifications Pages