



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401-1927
Telephone (612) 330-5500

October 22, 1992

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Potential Damage to Safe Shutdown Equipment
from an Elevator Failure During a Seismic Event

The voluntary Licensee Event Report for this occurrence is attached. Please contact us if you require further information.



Thomas M. Parker

Manager
Nuclear Support Services

c: Regional Administrator - III NRC
Sr Resident Inspector, NRC
NRP Project Manager, NRC
State of Minnesota,
Attn: Kris Sanda

Attachment

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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|---|--------|-----------|--|-------------------|-----------------|--------|------------------|--------------|-------------------|---|---------------------|-----------|------------------|----------------------|--|--------|-----------|--------------|-------------------|---|---|
| FACILITY NAME (1) Monticello Nuclear Generating Plant | | | | | | | | | | DOCKET NUMBER (2) 0 5 0 0 0 2 6 3 | | | | PAGE (3) 1 OF 0 3 | | | | | | | |
| TITLE (4) Potential Damage to Safe Shutdown Equipment from an Elevator Failure During a Seismic Event | | | | | | | | | | | | | | | | | | | | | |
| EVENT DATE (5) | | | LER NUMBER (6) | | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | | | | | | | | | | |
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | | | DOCKET NUMBER(S) | | | | | | | | |
| 0 | 9 | 2 | 2 | 9 | 2 | 0 | 1 | 1 | 0 | 0 | 1 | 0 | 2 | 2 | 9 | 2 | 0 | 5 | 0 | 0 | 0 |
| OPERATING MODE (9) N | | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11) | | | | | | | | | | | | | | | | | | |
| POWER LEVEL (10) 1 0 0 | | | 20.402(b) | | | | 20.406(c) | | | | 50.73(a)(2)(iv) | | | | 73.71(b) | | | | | | |
| | | | 20.405(a)(1)(i) | | | | 50.36(c)(1) | | | | 50.73(a)(2)(v) | | | | 73.71(c) | | | | | | |
| | | | 20.405(a)(1)(ii) | | | | 50.36(c)(2) | | | | 50.73(a)(2)(vi) | | | | <input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A) | | | | | | |
| | | | 20.405(a)(1)(iii) | | | | 50.73(a)(2)(i) | | | | 50.73(a)(2)(vii)(A) | | | | | | | | | | |
| | | | 20.405(a)(1)(iv) | | | | 50.73(a)(2)(ii) | | | | 50.73(a)(2)(vii)(B) | | | | | | | | | | |
| | | | 20.405(a)(1)(v) | | | | 50.73(a)(2)(iii) | | | | 50.73(a)(2)(ix) | | | | | | | | | | |
| LICENSEE CONTACT FOR THIS LER (12) | | | | | | | | | | | | | | | | | | | | | |
| NAME Steve Hammer, Supt Turbine Systems Engineering | | | | | | | | | | TELEPHONE NUMBER AREA CODE 6 1 2 2 9 5 - 1 3 0 0 | | | | | | | | | | | |
| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | | | | | | | | | | | | | |
| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | | |
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| SUPPLEMENTAL REPORT EXPECTED (14) | | | | | | | | | | EXPECTED SUBMISSION DATE (15) | | MONTH | | DAY | | YEAR | | | | | |
| <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) | | | | | | | | | | <input checked="" type="checkbox"/> NO | | | | | | | | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During the seismic evaluation for the installation of a new elevator in the Reactor Building, it was determined that the existing elevator could have damaged safe shutdown equipment during a design bases seismic event. The cause of this event was that the original design bases did not require seismic qualification of the elevator. The existing elevator was immediately removed and the new elevator will be installed to meet seismic criteria.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20545-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

| FACILITY NAME (1) | DOCKET NUMBER (2) | LER NUMBER (6) | | | PAGE (3) |
|-------------------------------------|-------------------|----------------|-------------------|-----------------|----------|
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | |
| Monticello Nuclear Generating Plant | 05000 263 | 92 | - 011 - | 00 | 2 OF 3 |

TEXT (If more space is required, use additional copies of NRC Form 866A. (17)

DESCRIPTION

On September 22, 1992, with the plant operating at 100% of rated thermal power, it was determined by plant engineers that the Reactor Building (EIIS System: NG) elevator (EIIS Component: ELEV) could have failed in such a manner as to damage safety related equipment.

The Reactor Building elevator, which was installed as original equipment during initial plant construction, had been removed and installation of the new elevator was pending completion of a seismic evaluation. The evaluation determined that a postulated seismic event for the existing elevator could have resulted in an unacceptable interaction with seismic Class I equipment. The postulated event was a failure of the elevator upper structure during a design bases seismic event, such that the car and counter weight could fall with enough force to damage the 935 foot elevation Reactor Building floor. The car, counter weight and/or concrete of the damaged floor could strike components of the Division I Residual Heat Removal (EIIS System: RO) and Core Spray (EIIS System: BM) systems preventing these systems from performing their functions.

CAUSE

The cause of this event was that the original design bases did not require seismic qualification of the elevator.

ANALYSIS

Both Emergency Diesel Generators (EIIS Comp: DG), Division II Residual Heat Removal and Core Spray systems, High Pressure Coolant Injection (EIIS System: BJ) and the Safety Relief Valves (EIIS Comp: RV) were available to achieve safe shutdown conditions.

Because of the availability of redundant equipment to achieve safe shutdown there were no consequences to the health and safety of the public.

This event is not reportable under 10 CFR Part 50, Section 50.73, but is being reported voluntarily because of the potential generic interest.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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| Monticello Nuclear Generating Plant | | 05000 263 | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | 3 OF 3 |
| | | | | 92 | 011 | 00 | |

TEXT (If more space is required, use additional copies of NRC Form 305A) (17)

CORRECTIVE ACTIONS

1. All original elevator equipment has been removed.
2. A new elevator will be installed which meets seismic system interface requirements to assure no impact on Class 1 equipment.

ADDITIONAL INFORMATION

Failed Components:

None

Previous Similar Event:

None