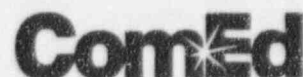


Commonwealth Edison Company  
Quad Cities Generating Station  
22710 200th Avenue North  
Cordova, IL 61242-9740  
Tel 309-654-2241



LWP-96-052

July 10, 1996

U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Attention: Document Control Desk

Reference: Quad Cities Nuclear Power Station  
Docket Number 50-254, DPR-29, Unit One

Enclosed is Licensee Event Report (LER) 96-010, Revision 00, for Quad Cities Nuclear Power Station.

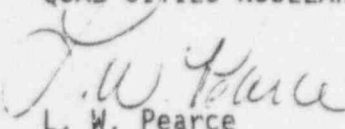
This report is submitted in accordance with the requirements of the Code of Federal Regulations, Title 10, Part 50.73(a)(2)(ii)(B). The licensee shall report any event or condition that resulted in the condition of the nuclear power plant, including its principal safety barriers, being seriously degraded, or that resulted in the nuclear plant being in a condition that was outside the design basis of the plant.

All corrective actions have been completed.

If there are any questions or comments concerning this letter, please refer them to Nick Chrissotimos, Regulatory Assurance Supervisor at 309-654-2241, ext. 3100.

Respectfully,

COMMONWEALTH EDISON  
QUAD CITIES NUCLEAR POWER STATION

  
L. W. Pearce  
Station Manager

LWP/NC/plm

Enclosure

cc: P. Piet, Licensing, ComEd  
C. Miller, Senior Resident Inspector, Quad Cities  
INPO Records Center  
H. J. Miller, Regional Administrator, Region III  
R. J. Singer, Mid American Energy Company  
D. C. Tubbs, Mid American Energy Company  
R. M. Pulsifer, Project Manager, NRR

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Form Rev. 2.0

FACILITY NAME (1)  Quad Cities Unit One	DOCKET NUMBER (2)  0   5   0   0   0   2   5   4	LER NUMBER (6)						PAGE (3)  2   OF   0   4
		Year		Sequential Number		Revision Number		
		9   6	-	0   1   0	-	0   0		

TEXT Energy Industry Identification System (EIIIS) codes are identified in the text as [XX]

**PLANT AND SYSTEM IDENTIFICATION:**

General Electric - Boiling Water Reactor - 2511 MWt rated core thermal power.

**EVENT IDENTIFICATION:** The gallery steel above the Primary Containment Equipment Hatch on Units 1 and 2 could have been degraded during a postulated Seismic Event due to inadequate design configuration control practices.

**A. CONDITIONS PRIOR TO EVENT:**

Unit: One	Event Date: June 11, 1996	Event Time: 2002
Reactor Mode: 1	Mode Name: Shutdown	Power Level: 00%

This report was initiated by Licensee Event Report 254\96-010.

SHUTDOWN (1) - In this position, a reactor scram is initiated, power to the control rod drives is removed, and the reactor protection trip systems have been deenergized for 10 seconds prior to permissive for manual reset.

**B. DESCRIPTION OF EVENTS:**

On June 11, 1996 a design change was being evaluated to add a pipe support to the gallery steel over the Unit 1 Primary Containment (Drywell) equipment hatch (X-1 penetration). A discrepancy was noted between the gallery steel drawings and the "as installed" configuration. The installed gallery steel configuration lacked several required cross bracing members. The purpose of the cross bracing is to qualify the gallery steel for seismic loading. Further research found the same condition existed on Unit 2. A subsequent Emergency Notification System (ENS) notification was made at 2002 on June 11, 1996. An expanded review found that the supporting calculation was written for Quad Cities Units 1 and 2 and Dresden Units 2 and 3. Dresden Station was notified and they confirmed their gallery steel had proper cross bracing installed.

**C. CAUSE OF EVENT:**

The root cause of this event was a deficiency in the original Design Configuration Control Process. This process did not enforce required plant changes to be implemented. The current Design Configuration Control Process precludes this type of discrepancy from occurring.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Form Rev. 2.0

FACILITY NAME (1)  Quad Cities Unit One	DOCKET NUMBER (2)  0   5   0   0   0   2   5   4	LER NUMBER (6) <table border="1"> <tr> <td data-bbox="941 161 1039 215">Year</td> <td data-bbox="1039 161 1096 215"></td> <td data-bbox="1096 161 1274 215">Sequential Number</td> <td data-bbox="1274 161 1364 215"></td> <td data-bbox="1364 161 1453 215">Revision Number</td> <td data-bbox="1453 161 1534 215"></td> </tr> <tr> <td>9</td> <td>6</td> <td>-</td> <td>0</td> <td>1</td> <td>0</td> </tr> </table>	Year		Sequential Number		Revision Number		9	6	-	0	1	0	PAGE (3)  3   OF   0   4
Year		Sequential Number		Revision Number											
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TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

**D. SAFETY ANALYSIS:**

During a postulated seismic event the failure of the gallery could potentially effect the Primary Containment (largest potential breach is a 10 inch dia. line), "A" Loop Residual Heat Removal (RHR)(potential failure of 2 valves), High Pressure Coolant Injection (HPCI)(potential failure of Differential Pressure instruments causing isolation), Recirculation Pumps (potential loss of power) and Drywell/Torus Vacuum Breakers (potential electrical shorts could hold vacuum breakers open). All these system/components have redundant systems/components which are available for safe shutdown of the Unit or are not required for safe shutdown of the Unit.

The safety significance is minimal in that redundant systems/components were available to bring the reactor to cold shutdown.

**E. CORRECTIVE ACTIONS:**

Corrective Actions Completed:

- 1) The gallery cross bracing was installed in both Units to achieve seismic qualification.
- 2) A review of Unit 1 high impact systems as determined by the Probabilistic Risk Assessment Program was conducted. The review identified galleries in the vicinity of equipment and components that are required to mitigate the consequences of design basis accident. These galleries were walked down to investigate possible discrepancies between "as designed" and "as built" configurations. One minor discrepancy which was not structurally significant was noted.
- 3) Station gallery drawings were reviewed to identify galleries revised from the original construction configuration. This review identified there was no potential to have a similar discrepancy between "as designed" and "as built" gallery configurations because structural enhancement revisions were not found.
- 4) Galleries were randomly selected from both Units 1 and 2 and these galleries were walked down to verify proper configuration. The randomly selected galleries matched the "as built" in all but two cases where minor discrepancies were found. The discrepancies were not structurally significant.

Based on the investigation performed, the gallery cross bracing over the X-1 penetration appears to be an isolated case.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Form Rev. 2.0

FACILITY NAME (1)  Quad Cities Unit One	DOCKET NUMBER (2)  0   5   0   0   0   2   5   4	LER NUMBER (6)			PAGE (3)  4   OF   0   4
		Year	Sequential Number	Revision Number	
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**F. PREVIOUS EVENTS:**

There have been no similar LERs since the beginning of 1994 regarding "as designed" versus "as built" problems.

**G. COMPONENT FAILURE DATA:**

Not applicable.