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Docket Nos.: 52-025

ND-20-0195
10 CFR 52.99(c)(1)

U.S. Nuclear Regulatory Commission
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Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3
ITAAC Closure Notification on Completion of ITAAC 2.1.03.02a [Index Number 69]

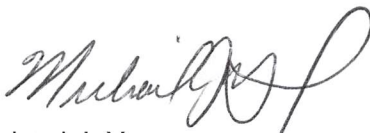
Ladies and Gentlemen:

In accordance with 10 CFR 52.99(c)(1), the purpose of this letter is to notify the Nuclear Regulatory Commission (NRC) of the completion of Vogtle Electric Generating Plant (VEGP) Unit 3 Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Item 2.1.03.02a [Index Number 69], for verifying the Reactor System (RXS) accommodates the fuel and control assemblies. The closure process for this ITAAC is based on the guidance described in NEI-08-01, "Industry Guideline for the ITAAC Closure Process under 10 CFR Part 52", which is endorsed by the NRC in Regulatory Guide 1.215.

This letter contains no new NRC regulatory commitments. Southern Nuclear Operating Company (SNC) requests NRC staff confirmation of this determination and publication of the required notice in the Federal Register per 10 CFR 52.99.

If there are any questions, please contact Tom Petrak at 706-848-1575.

Respectfully submitted,



Michael J. Yox
Regulatory Affairs Director Vogtle 3 & 4

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MJY/WLP/sfr

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To:

Southern Nuclear Operating Company/ Georgia Power Company

Mr. Peter P. Sena III (w/o enclosures)

Mr. D. L. McKinney (w/o enclosures)

Mr. M. D. Meier (w/o enclosures)

Mr. D. H. Jones (w/o enclosures)

Mr. G. Chick

Mr. M. Page

Mr. P. Martino

Mr. M. J. Yox

Mr. A. S. Parton

Ms. K. A. Roberts

Mr. T. G. Petrak

Mr. C. T. Defnall

Mr. C. E. Morrow

Mr. R.L. Beilke

Mr. S. Leighty

Ms. A. C. Chamberlain

Mr. J. C. Haswell

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cc:

Nuclear Regulatory Commission

Mr. W. Jones (w/o enclosures)

Mr. C. P. Patel

Mr. G. J. Khouri

Ms. S. E. Temple

Mr. N. D. Karlovich

Mr. A. Lerch

Mr. C. J. Even

Mr. B. J. Kemker

Ms. N. C. Coovert

Mr. C. Welch

Mr. J. Gaslevic

Mr. V. Hall

Mr. G. Armstrong

Ms. T. Lamb

Mr. M. Webb

Mr. T. Fredette

Mr. C. Weber

Mr. S. Smith

Mr. C. Santos

Mrs. M. Bailey

Mr. S. Rose

Mr. B. Davis

Mr. J. Vasquez

Mr. J. Eargle

Ms. K. Carrington

Mr. P. Heher

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Mr. M. King
Mr. E. Davidson

Oglethorpe Power Corporation

Mr. R. B. Brinkman
Mr. E. Rasmussen

Municipal Electric Authority of Georgia

Mr. J. E. Fuller
Mr. S. M. Jackson

Dalton Utilities

Mr. T. Bundros

Westinghouse Electric Company, LLC

Dr. L. Oriani (w/o enclosures)
Mr. D. C. Durham (w/o enclosures)
Mr. M. M. Corletti
Mr. Z. S. Harper
Mr. J. L. Coward

Other

Mr. J. E. Hesler, *Bechtel Power Corporation*
Ms. L. Matis, *Tetra Tech NUS, Inc.*
Dr. W. R. Jacobs, Jr., Ph.D., *GDS Associates, Inc.*
Mr. S. Roetger, *Georgia Public Service Commission*
Ms. S. W. Kernizan, *Georgia Public Service Commission*
Mr. K. C. Greene, *Troutman Sanders*
Mr. S. Blanton, *Balch Bingham*

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**Southern Nuclear Operating Company
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Enclosure**

**Vogtle Electric Generating Plant (VEGP) Unit 3
ITAAC Closure Notification on Completion of ITAAC 2.1.03.02a [Index Number 69]**

ITAAC Statement

Design Commitment:

2.a) The reactor upper internals rod guide arrangement is as shown in Figure 2.1.3-1.

2.b) The control assemblies (rod cluster and gray rod) and drive rod arrangement is as shown in Figure 2.1.3-2.

Inspections, Tests, Analyses:

Inspection of the as-built system will be performed.

Inspection of the as-built system will be performed.

Acceptance Criteria:

The as-built RXS will accommodate the fuel assembly and control rod drive mechanism pattern shown in Figure 2.1.3-1.

The as-built RXS will accommodate the control assemblies (rod cluster and gray rod) and drive rod arrangement shown in Figure 2.1.3-2.

ITAAC Determination Basis

This ITAAC requires inspections be performed of the as-built Reactor System (RXS) to verify the reactor upper internals rod guide pattern is as shown in the Combined License (COL) Appendix C Figure 2.1.3-1 (Attachment A) and the control assemblies (rod cluster and gray rod) and drive rod arrangement are as shown in COL Appendix C Figure 2.1.3-2 (Attachment B).

Inspections of the as-built RXS were performed to verify that the reactor upper internals rod guide pattern is as shown in Figure 2.1.3-1 (Attachment A) and that the control assemblies (rod cluster and gray rod) and drive rod arrangement are as shown in Figure 2.1.3-2 (Attachment B). Additionally, a review of inspection records verified that the as-built dimensions of the Reactor Vessel Internals are as shown in COL Appendix C Table 2.1.3-4 (Attachment C).

Dimensional inspections were performed at the vendor's facility. Due to the nature of the manufacturing process of the reactor vessel head and internals, it was necessary to verify measurements were within the ITAAC acceptable ranges prior to shipment and final installation. Completing these measurements at the vendor's facility meets the definition of "as-built inspections" per NEI 08-01, Section 9.5, "As-built" Inspections (Reference 1).

The inspection results are documented in the Principle Closure Documents (References 2 and 3) supporting the ITAAC 2.1.03.02a Completion Package (Reference 4) and confirm that the Unit 3 as-built RXS will accommodate the fuel assembly and control rod drive mechanism pattern shown in Figure 2.1.3-1 and will accommodate the control assemblies (rod cluster and gray rod) and drive rod arrangement shown in Figure 2.1.3-2. References 2 and 3 are available for NRC inspection as part of the ITAAC 2.1.03.02a Completion Package (Reference 4).

ITAAC Finding Review

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

ITAAC Completion Statement

Based on the above information, SNC hereby notifies the NRC that ITAAC 2.1.03.02a was performed for VEGP Unit 3 and that the prescribed acceptance criteria are met.

Systems, structures, and components verified as part of this ITAAC are being maintained in their as-designed, ITAAC compliant condition in accordance with approved plant programs and procedures.

References (available for NRC inspection)

1. NEI 08-01, Industry Guideline for the ITAAC Closure Process under 10 CFR Part 52
2. SV3-MI01-VQQ-002, Revision 9, AP1000 Reactor Vessel Internals (QR & C of C)
3. LTR-ARIDA-20-1, Revision 0, Use of Vogtle Unit 3 As-Built Reactor Vessel Internals Data to Support Dimensions J and K as Related to ITAAC Item 2.1.03.02a (Index Number 69)
4. 2.1.03.02a-U3-CP-Rev0, ITAAC Completion Package

Attachment A: COL Appendix C Figure 2.1.3-1

The Reactor Upper Internals Rod Guide Arrangement (Attachment A) was verified to accommodate the fuel assembly and control rod drive mechanism pattern per inspection of SV3-MI01-VQQ-002, page 2261, Section A-A (Reference 2).

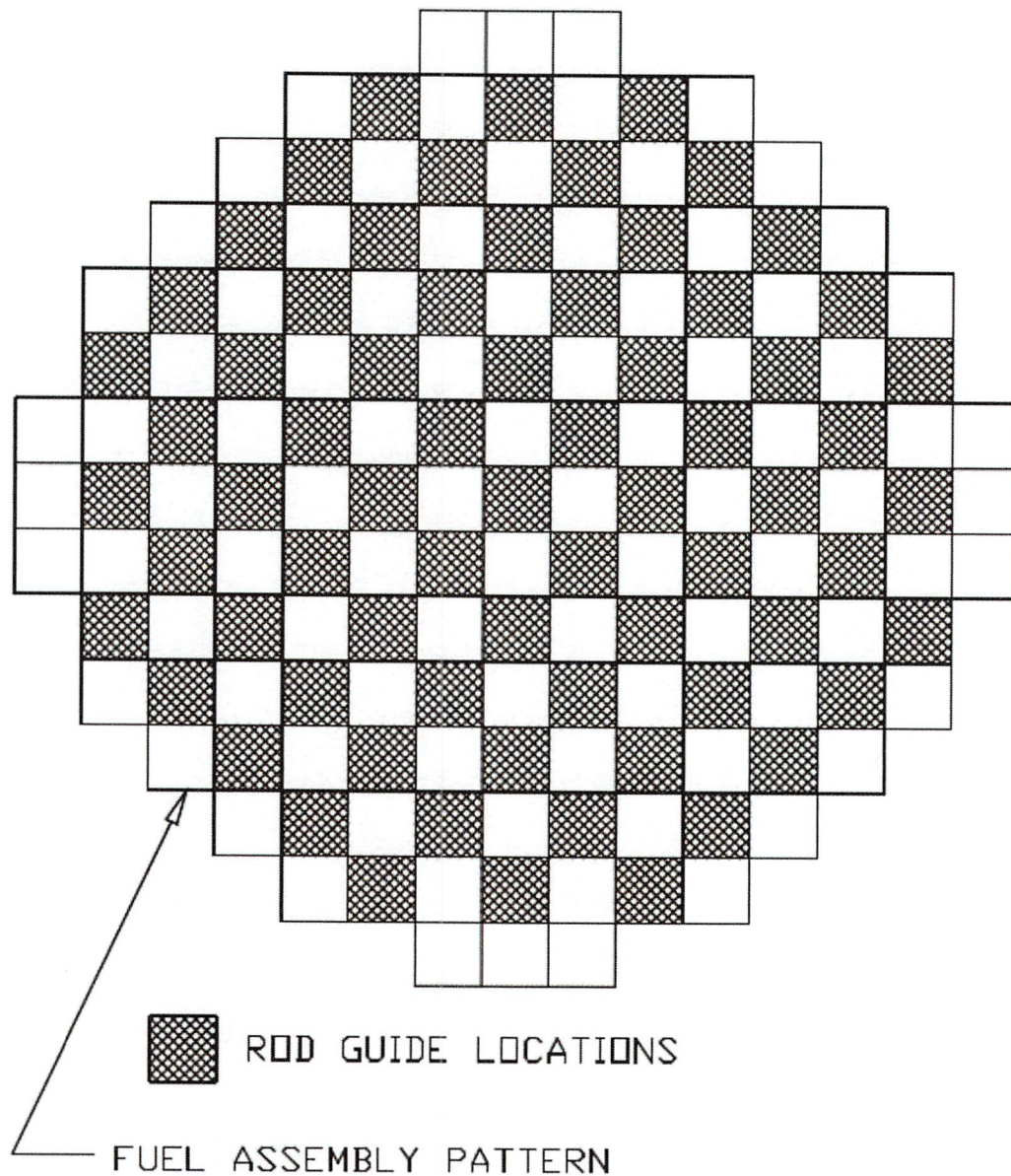


Figure 2.1.3-1
Reactor Upper Internals Rod Guide Arrangement

Attachment B: COL Appendix C Figure 2.1.3-2

Labeled items and configurations were verified per inspection of Reference 2. Dimensions J and K were verified per inspection of Reference 3 (see Table 2.1.3-4, Attachment C).

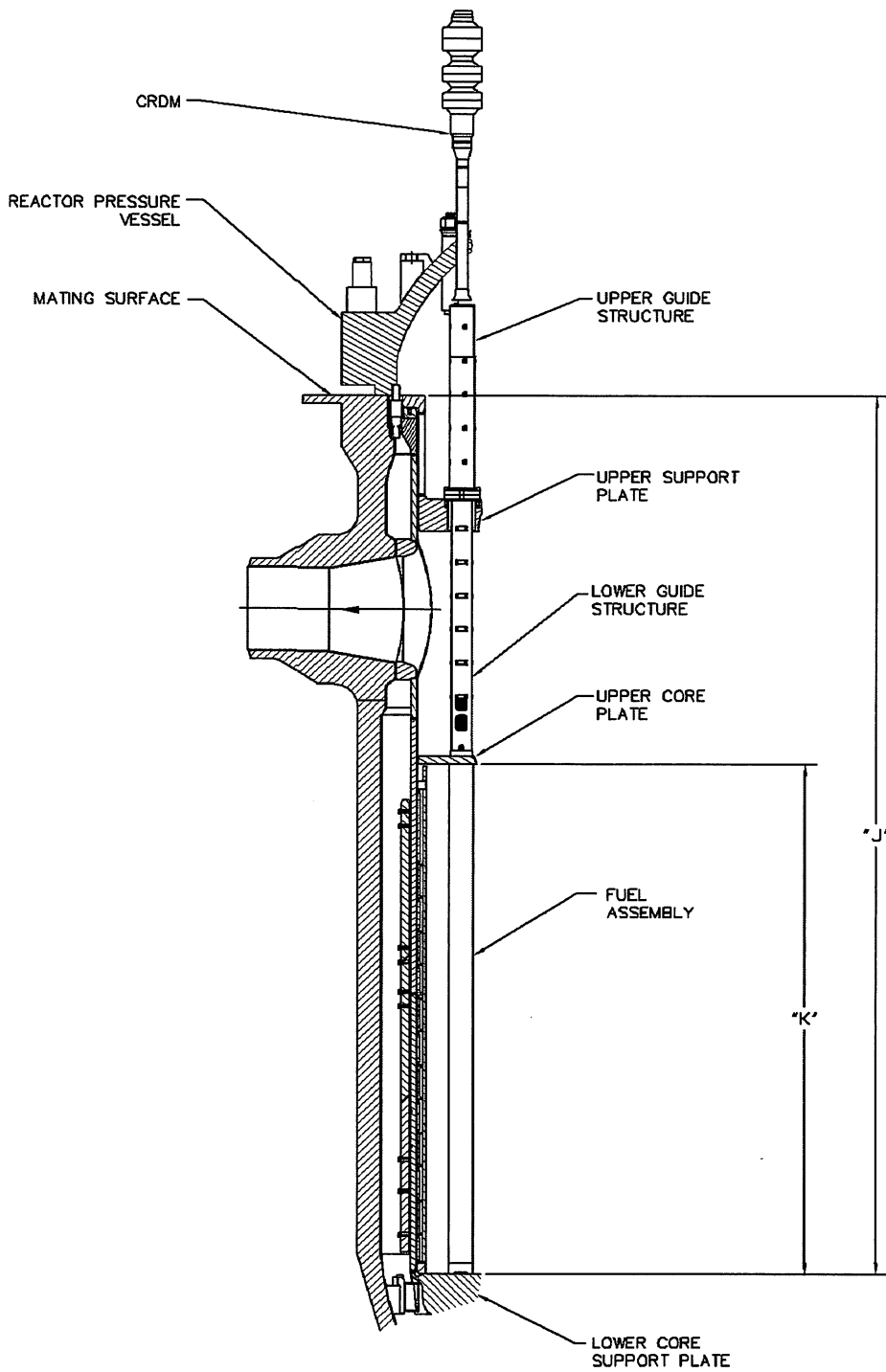


Figure 2.1.3-2
Rod Cluster Control and Drive Rod Arrangement

Attachment C

Table 2.1.3-4* Key Dimensions and Acceptable Variations of the Reactor Vessel Internals (Figure 2.1.3-2)				
Description*	Dimension or Elevation* (inches)	Nominal Value* (inches)	Acceptable Variation* (inches)	QR & C of C Values – Min. to Max** (inches)
Elevation from RV mating surface to top of lower core support plate	J	327.3	+0.50/-0.50	327.300 – 327.305
Separation distance between bottom of upper core plate and top of lower core support with RV head in place	K	189.8	+0.20/-0.20	189.786 – 189.804

* - Excerpt from COL Appendix C Table 2.1.3-4

** - Min to Max dimensions taken from Westinghouse letter LTR-ARIDA-20-1 (Reference 3)