GPU Nuclear

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July 11, 1996
6710-96-22[illegible]

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for June 1996

Enclosed are two copies of the June 1996 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

J. Knubel
Vice President and Director, TMI

WGH

cc: Administrator, Region I
TMI Senior Resident Inspector
96001

IE24

160074

OPERATIONS SUMMARY
June 1996

The plant entered the month operating at 100% power until 29 June when power was reduced to approximately 50% due to Main Condenser tube leakage. After repairs to the condenser were completed, the plant returned to 100% power on 30 June. Net unit electrical output averaged approximately 774 MWe during June.

MAJOR SAFETY RELATED MAINTENANCE

There were no major safety related maintenance items completed during the month. The following description of Main Condenser maintenance is provided for information.

Main Condenser CO-C-1B

The 'B' side of the Main Condenser CO-C-1B was removed from service during the month of June due to tube leakage. The tube bundle was leak tested using SF_6 gas. After identifying one tube as leaking, Eddy Current testing was performed on 25 tubes and the identified leaker. The Eddy Current test identified three tubes with >60% through wall indications. These three tubes and the leaking tube were plugged. The affected side of the condenser was operated and tested with SF_6 to confirm the elimination of tube leakage prior to power escalation. When the post plugging testing indicated no further leakage, CO-C-1B was closed and returned to service. Actions were then initiated to return the plant to 100% power operation.

OPERATING DATA REPORT

DOCKET NO. 50-289
DATE July 11,1996
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

OPERATING STATUS

1. UNIT NAME: THREE MILE ISLAND UNIT 1
2. REPORTING PERIOD: JUNE 1996
3. LICENSED THERMAL POWER: 2568
4. NAMEPLATE RATING (GROSS MWe): 872
5. DESIGN ELECTRICAL RATING (NET MWe): 819
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe) 834
7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____
10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	720.0	4367.0	191352.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	720.0	4367.0	113910.1
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	720.0	4367.0	112748.3
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1,802,119.7	11,159,706.2	277,274,632.2
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	590,063.0	3,736,789.0	93,190,263.1
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	557,033.0	3,530,810.0	87,574,214.1
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	58.9
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	58.9
21. UNIT CAPACITY FACTOR	(USING MDC NET)	98.4	102.9	58.2
22. UNIT CAPACITY FACTOR	(USING DER NET)	94.5	98.7	55.9
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	35.0
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH): _____
25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE July 11, 1996
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: JUNE

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
1	807	17	791
2	806	18	792
3	809	19	795
4	804	20	795
5	802	21	791
6	801	22	790
7	796	23	797
8	790	24	796
9	791	25	791
10	789	26	800
11	792	27	799
12	795	28	788
13	794	29	335
14	792	30	594
15	793	31	#N/A
16	794		

DOCKET NO. 50-289
UNIT NAME TMI-1
DATE July 11, 1996
COMPLETED BY W. G. Heysek
TELEPHONE (717) 948-8191

REPORT MONTH June 1996

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report#	System Code [4 & 5]	Component Code [5 & 5]	Cause & Corrective Action to Prevent Recurrence
96-01	6-29-96	F	0	5	4	None	SG	Cond	During the early morning of June 28, the plant developed a chemistry anomaly which indicated the presence of condenser tube leakage. At 00:00 on June 29 plant power was reduced to 50% for the purpose of performing tube leak testing and repair. Following the appropriate tube plugging, the plant returned to full power operation at 14:12 on June 30.

1
F Forced
S Scheduled

2
Reason
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1 same source

6 Actually used exhibits F & H NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**

2. Scheduled date for next refueling shutdown: **September 5, 1997**

3. Scheduled date for restart following current refueling: **NA**

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **Yes. To support GPU Nuclear plans to do independent reload analyses for Cycle 12 as discussed in response to question 6 below, T.S. 6.9.5.2 would require revision to include references to the GPU Nuclear analysis methods applied to the reload.**

5. Scheduled date(s) for submitting proposed licensing action and supporting information: **A Technical Specification Change Request for the changes as discussed above would be submitted.**

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: **GPU Nuclear Letter 6710-96-2092, dated March 28, 1996 confirmed plans to perform independent reload design evaluations for Cycle 12, the next operation cycle, based on NRC approved methods described in GPU Nuclear Topical Reports TR-091 (core physics), TR-087 (core thermal hydraulics), TR-078 (FSAR safety analyses) and TR-092P (design and setpoints methodology) submitted to the NRC.**

   The NRC has issued an SER for TR-091 on February 21, 1996 and has recently indicated that Generic Letter 83-11 Supplement 1 is on hold. Accordingly NRC review and approval is needed for each of the three remaining GPU Nuclear Topical Reports. A meeting with NRC staff reviewers was held on May 30, 1996 during which the Cycle 12 reload analysis schedules were discussed. At this time, completion of the NRC review and issuance of NRC SERs by the following critical approval dates is achieveable:

TR-087	**8/1/96**
TR-078	**10/1/96**
TR-092P	**10/1/96**

   The GPU Nuclear Cycle 12 reload program and results are expected to be available for NRC review in the March to April 1997 time frame.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) **177** (b) **864**

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained. Phase II is expected to be started in 2002.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.