10CFR50.59

Boston Edison

Pilgrim Nuclear Power Station
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U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

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Report of Changes, Tests and Experiments
Performed at Pilgrim Nuclear Power Station

In accordance with 10CFR50.59(b), Boston Edison is submitting this report of the changes, tests and experiments at Pilgrim Nuclear Power Station for the period of January 1 through December 31, 1995.

A listing of all changes affecting the Final Safety Analysis Report (FSAR) completed in the reporting period is attached. Each listing contains a brief description, a reference to the relevant FSAR sections, and a reference to the supporting safety evaluation(s).

No tests or experiments were performed during the report period.

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HVO/MTL/pkk/radmisc/5059RPT

Attachment: Report of Changes, Tests and Experiments

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IE47 1/1

Replace All Piping Thermal Insulation in the Drywell

PDC 83-62
Safety Evaluation Numbers 2971, 1638 Rev. 1
FSAR Sections Affected: 4.8.5.1, 6.4.3, 14.5.3.1.3, F14.5-9, F14.5-10, F14.5-13

The change replaced all the piping thermal insulation in the drywell with Owens-Corning NUKON fiberglass insulation. This change was previously reported in Boston Edison letter dated July 25, 1986. Safety evaluation 1638 Rev. 1 was superseded by safety evaluation 2971 because an original General Electric report attached to the safety evaluation has been revised. Safety evaluation 2971 also expanded the background information related to the issue of pump NPSH and the Pilgrim design basis.

The design change reviewed by this safety evaluation does not constitute an unreviewed safety question.

Upgrade 480V AC Station Service Substation No. 1

PDC 85-84
Safety Evaluation Number 2498
FSAR Sections Affected: Figure 8.2-1

This modification replaced the existing transformer and its associated 480V AC distribution system with a new substation.

The replacement system does not have a safety-related function. The new system is an improvement over the old system and is not located near any safety-related equipment; therefore, this change did not constitute an unreviewed safety question.

Augmented Fuel Pool Cooling and Purification

PDC 91-23
Safety Evaluation 2576
FSAR Sections Affected: 4.8.5.6, 10.4.3

This modification revised the reactor cavity cleanup mode of the residual heat removal and fuel pool cooling and demineralizer systems. This modification allows effluent from the fuel pool filter and demineralizer to flow continuously to either the spent fuel pool or the reactor cavity to facilitate spent fuel pool cleanup and reactor cavity cooling during refueling.

The modification did not compromise the ability of the affected safety-related components from performing their safety-related functions; therefore, there was no unreviewed safety question involved.

Removal of Fire Dampers CR-6 and CR-7

PDC 91-68
Safety Evaluation 2654
FSAR Figure Affected: F10.9-6

This modification removed fire dampers CR-6 and CR-7 that were located in the control room high efficiency air filtration system (CRHEAFS) because they had the potential to impact CRHEAFS if they were to close inadvertently. There was also potential degradation of the cinch straps, and they may not have been capable of holding the dampers open in a seismic event. Removal of these dampers eliminated the concerns and did not adversely affect the integrity of the fire barrier. This change did not involve an unreviewed safety question.

Cable Replacement From D16 to MCC D7

PDC 93-46
Safety Evaluation 2811
FSAR Section Affected: F8.6-1

This modification replaced the cable from D16 to MCC D7 to improve the voltage at the motor terminals for certain motor operated valves (MOV) fed from MCC D7. Replacing this feeder cable reduced the scope of MOV modifications required to improve the design margin under the Generic Letter 89-10 program.

This change did not involve an unreviewed safety question.

Replacement of Turbine Building Trucklock Rollup Door

PDC 94-08
Safety Evaluation 2058
FSAR Figures Affected: F12.1-2, F12.1-3, F12.1-11, F12.2-7, F12.3-2

This change removed the existing 16' wide rollup door in the turbine building trucklock and replaced it with a new 20' wide rollup door.

Replacing the trucklock door did not adversely affect safety-related structures or equipment and did not involve an unreviewed safety question.

Modification to MO1001-50 for Generic Letter 89-10

PDC 94-18E
Safety Evaluation 2861
FSAR Section Affected: Table 5.2-4

This modification involved installing a new larger valve operator motor, revising the four rotor limit switches, added a new torque switch, and replacement of the thermal overload relay and overload heaters. A Liberty Quicktest cable assembly was also installed in the actuator to allow diagnostic testing of the motor operated valve without opening the actuator switch compartment. These modifications were part of the Generic Letter 89-10 program.

This change did not involve a unreviewed safety question. The safety related function of the valve remained qualified for plant design bases loads after completion of this modification.

Actuator Replacement for MO1001-47

PDC 94-18K
Safety Evaluation 2888
FSAR Section Affected: Table 5.2-4

This modification included installation of a new larger valve operator motor and replacement of the thermal overload heater as part of the Generic Letter 89-10 program.

This modification improved the ability of MO1001-47 to perform its safety function and did not involve an unreviewed safety question.

Spent Fuel Control Rod Blade Storage

PDC 94-26
Safety Evaluation 2818
FSAR Section Affected: 10.3.4.1

This change provided brackets, hardware and cables to individually suspend control rod blades from the fuel pool curb. This change provides open space on the fuel pool floor for spent fuel racks and waste containers.

This change did not constitute an unreviewed safety question.

Augmented Fuel Pool Cooling - Phase 2 Modifications

PDC 94-37
Safety Evaluation Number: 2886
FSAR Sections Affected: 4.8.5.5, 4.8.5.6, 10.4.3

This change modified the residual heat removal/fuel pool cooling intertie to improve the operation of the augmented fuel pool cooling mode 1 and establish a new augmented fuel pool cooling mode 2.

This modification does not compromise the ability of the affected safety related components to perform their safety functions, and there is no unreviewed safety question involved.

Reactor Shroud Repair

PDC 94-43
Safety Evaluation 2926
FSAR Sections Affected: 3.3.4, 3.3.4.1.1, 3.3.6.7.1, 3.3.6.9, F3.3-10, C.3.4.2, TC.3-7

This change provided a permanent preemptive repair to the core shroud in the Pilgrim reactor pressure vessel in response to Generic Letter 94-03. The repair structurally replaced all of the shroud circumferential girth welds by installing a vertical tie rod and lateral spring arrangement at each of four locations on the outside of the shroud.

The shroud repair did not constitute an unreviewed safety question.

Salt Service Water Pump Cubicle Exhaust Fans

PDC 95-12
Safety Evaluation Number: 2911
FSAR Section Affected: 10.9.3.7.2

This change replaced the air operated inlet dampers for the Salt Service Water cubicle exhaust fans with manually operated dampers. No unreviewed safety question was identified by this safety evaluation.

Annual Revision of the Emergency Plan

Safety Evaluation 2950
FSAR Section Affected: Appendix N

This revision to the Pilgrim Station Emergency Plan reflects those changes previously approved and administrative changes that have been identified since the last revision of the plan. This change did not involve an unreviewed safety question.

Surveillance Requirements For Fire Protection

Safety Evaluation 2976
FSAR Sections Affected: 10.8.4, 10.8.4.1.2, 10.8.4.2.2, 10.8.4.3.2, 10.8.4.5.2, 10.8.4.6.2

This change incorporated changes to the NFPA codes and standards for fire protection. In particular, the test frequencies for detectors, pumps, batteries, valves, fire hoses and penetration seals have been modified.

No unreviewed safety questions were identified in this evaluation.

Salt Service Water Temperature of 75°F

Safety Evaluation: 2983
FSAR Sections Affected: 1.6, 4.8, 5.2, 10.5, 14.5, 14.7

This change resulted from a new plant safety analyses performed using a constant salt service water system injection temperature of 75°F. This evaluation also documented the proposed changes to the FSAR which incorporated results from the design verification consistent with the scope of information and level of detail in the current FSAR. The new analyses presented in this evaluation supplement the current analyses.

This change did not involve an unreviewed safety question.

Building Waterproof Membrane Bypass Seepage

Safety Evaluation Number: 2986
FSAR Section Affected: 12.2.4.4.3

This evaluation clarifies the current FSAR descriptions of the waterproof membrane feature of the reactor building, turbine building and radwaste building. The clarification states the waterproof membrane systems are designed to prevent or minimize groundwater in-leakage.

This change did not involve an unreviewed safety question.