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Senior Vice President

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10CFR50.73

October 9, 1992
NRC-92-0104

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

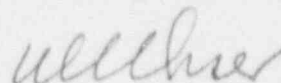
Reference: Fermi 2
NRC Docket No. 50-341
NRC License No. NPF-43

Subject: Licensee Event Report (LER) No. 92-007

Please find enclosed LER No. 92-007, dated October 9, 1992,
for a reportable event that occurred on September 13, 1992.
A copy of this LER is also being sent to the Regional
Administrator, I NRC Region III.

If you have any questions, please contact Barbara Siemasz,
Senior Compliance Engineer, at (313) 586-1683.

Sincerely,



Enclosure: NRC Forms 366, 366A

cc: T. G. Colburn
A. B. Davis
M. P. Phillips
S. Stasek
P. L. Torpey

Wayne County Emergency
Management Division

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Fernald 2										DOCKET NUMBER (2) 0 5 0 0 0 3 4 1 1 OF 0 3										PAGE (3) 1 OF 0 3	
TITLE (4) Exceeded Technical Specification Allowable Leakage for Containment Isolation Valves																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)									
0 9	1 3	9 2	9 2	0 0 7	0 0	1 0	0 9	9 2				0 5 0 0 0									
OPERATING MODE (9) 4		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																			
POWER LEVEL (10) 10		20.405(a)(i)(i)				20.405(a)(i)(ii)				50.73(a)(2)(iv)				73.71(b)							
		20.405(a)(i)(iii)				50.38(a)(1)				50.73(a)(2)(v)				73.71(c)							
		20.405(a)(i)(iv)				50.38(a)(2)				50.73(a)(2)(vi)				OTHER (Specify - Abstract only - and in Text NRC Form 365A)							
		20.405(a)(i)(v)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)											
		20.405(a)(i)(vi)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
		20.405(a)(i)(vii)				50.73(a)(2)(iii)				50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)												TELEPHONE NUMBER									
NAME Barbara Siemasz, Senior Compliance Engineer												AREA CODE 3 1 3 5 8 6 - 1 6 8 3									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC												
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input type="checkbox"/> NO		1	2	0 2 9 2							
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)																					

Periodic leakage rate testing of primary containment isolation valve and penetrations is performed in accordance with the requirements of Technical Specification (TS) 3.6.1.2 and 10 CFR 50, Appendix J. During the performance of this testing, several valves have exceeded their administrative allowable leakage rate and their combined leakage exceeds the limits as defined in TS 3.6.1.2.b.

Containment isolation valves that exceed their individual administrative allowable leakage rate have had work requests generated to repair or rework them as conditions dictate.

A full report will be provided in a supplement to this Licensee Event Report to be submitted 30 days after completion of the Type B and C testing/retesting.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Fermi 2	DOCKET NUMBER (2) 0 5 0 0 0 3 4 1 9 2	LER NUMBER (6)		PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
			0 0 7	0 1 0	0 1 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Initial Conditions:

Operational Condition: 4 (Cold Shutdown)
Reactor Power: 0%
Reactor Pressure: 0 psig
Reactor Temperatures: 112 degrees Fahrenheit

Description of Event:

On September 12, 1992, Local Leak Rate Testing (LLRT) of the Primary Containment Isolation Valves (ISV) and penetrations was initiated in accordance with Technical Specification (TS) 3/4.6.1.2, "Primary Containment Leakage", and 10 CFR 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water Cooled Power Reactors." As of October 5, 1992, approximately 83% of the penetrations (Type B tests) and 84% of the isolation valves (Type C tests) have been tested. Included in the above completed tests are the initial Type C LLRTs on the Main Steam Isolation Valves (MSIVs).

TS 3.6.1.2.b requires the combined leakage rate of type B and Type C tests, except for leakage tests on the MSIVs and valves which are hydrostatically tested, shall be less than or equal to 0.6 La [178 standard cubic feet per hour (scfh)]. Not including MSIV leakage in the combined total (178 scfh) is an approved exemption from 10 CFR 50, Appendix J. As of October 5, 1992, all Type B penetration tests have passed. However, we have tested a sufficient number of valves to indicate we will not pass the type C test. The combined Technical Specification leakage rate limit of 178 scfh will be exceeded.

TS 3.6.1.2.c requires that the leakage rates for all four main steam lines, when tested at 25.0 psig, be less than or equal to 100 scfh. The LLRT of these lines indicate that the combined leakage rates exceed the Technical Specification limit. The outboard MSIVs B and C had the highest leakage rates of the four main steam lines tested.

Additionally, TS 3.6.1.2.d requires that a combined leakage rate for all containment isolation valves in hydrostatically tested lines be less than or equal to 5 gallons per minute (gpm) when tested at 1.10 Pa (62.2 psig). As of October 5, 1992, the total leakage rate of hydrostatically tested valves has not exceeded the 5 gpm Technical Specification limit.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NO.			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Cause of the Event:

Leakage of containment isolation valves is generally caused by normal degradation of valve components and/or contaminants on the valve seating surfaces. Since disassembly and analysis of each valve, as required, is ongoing, the specific cause(s) of the excessively leaking isolation valves will be included in a supplement to this LER.

Analysis of the Event:

Type B and C LLRT are performed to ensure the leak-tight integrity of penetrations and valves affecting the primary containment boundary. Excessive leakage discovered through testing is corrected to minimize potential degradation of the primary containment boundary between integrated leak rate testing periods.

A complete analysis of the consequences of exceeding these Technical Specification limits will be provided in a supplement to this LER following completion of the Type B and C testing/retesting.

Corrective Actions:

Containment isolation valves that exceed their individual administrative allowable leakage rate have had work requests generated to repair or rework them as conditions dictate. These valves will be retested to determine their leakage rates.

A full report will be provided in a supplement to this LER to be submitted 30 days after completion of the Type B and C testing/retesting.

Previous Similar Occurrences:

LER 86-011-01: "Excessive Leakage from MSIV", LER 88-008-01: "Leakage In Excess of the Allowable Found During LLRT", LER 89-021-01: "Local Leak Rate Testing Exceeds Tech. Spec. Limits" and LER 91-005-01: "Exceeded Tech Spec Allowable Limits for Local Leak Rate Testing".