

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT I										DOCKET NUMBER (2) 0 5 0 0 0 3 2 1					PAGE (3) 1 OF 0 2										
TITLE (4) REACTOR SHUTDOWN REQUIRED BY TECH. SPECS.																									
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
0	2	1	6	8	5	8	5	0	1	3	0	0	0	3	1	8	8	5	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																							
1		20.402(b)					20.406(c)					50.73(a)(2)(iv)					73.71(b)								
POWER LEVEL (10)		0 1 5					20.406(a)(1)(i)					50.73(a)(2)(v)					73.71(c)								
		20.406(a)(1)(ii)					50.73(a)(2)(vi)					50.73(a)(2)(vii)(A)					OTHER (Specify in Abstract below and in Text, NRC Form 365A)								
		20.406(a)(1)(iii)					50.73(a)(2)(vii)(B)					50.73(a)(2)(viii)													
		20.406(a)(1)(iv)					50.73(a)(2)(ix)					50.73(a)(2)(x)													
		20.406(a)(1)(v)					50.73(a)(2)(xi)					50.73(a)(2)(xii)													
LICENSEE CONTACT FOR THIS LER (12)																									
NAME										TELEPHONE NUMBER															
T. L. Elton, Acting Superintendent of Regulatory Compliance										9 1 2 3 6 7 1 7 8 5 1															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS															
X	A	D	S	E	A	L	B	5	8	0	Y														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO													
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																									
<p>On 02/16/85 at approximately 1830 CST, with the reactor in steady state operation at 2436 MWt (approximately 100% power), operations personnel determined that drywell floor drain total leakage had risen to 32 gallons per minute, which was in excess of the maximum leakage allowed by Tech. Specs. section 3.6.G.1.c. At 1840 CST, operations personnel began a reactor power reduction in preparation for tripping and isolating the "B" reactor recirculation pump (1B31-C001B), because it had been determined that its shaft seals were leaking.</p> <p>At approximately 1858 CST, operations personnel initiated an orderly shutdown, tripped and isolated the "B" reactor recirculation pump (1B31-C001B). Drywell floor drain leakage then decreased to approximately 6.0 gallons per minute.</p> <p>At approximately 2124 CST, operations personnel manually scrambled the reactor as part of an orderly shutdown.</p> <p>Subsequently, the "B" reactor recirculation pump's (1B31-C001B) shaft seal cartridge was rebuilt and reinstalled. The pump was then satisfactorily functionally tested, and reactor startup commenced on 02/22/85 at approximately 1205 CST.</p>																									
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
EDWIN I. HATCH, UNIT 1	0 5 0 0 0 3 2 1 8 5	—	0 1 3	—	0 0 0	2	OF 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

This 30 day LER is required by 10CFR50.73(a)(2)(i)(A) because this event shows that the reactor was shut down per Tech. Specs. section 3.6.J.2.

On 02/16/85 at approximately 1830 CST, with the reactor in steady state operation at 2436 MWt (approximately 100% power), operations personnel determined that drywell floor drain total leakage had risen to 32 gallons per minute, which was in excess of the 25 gallons per minute maximum leakage allowed by Tech. Specs. section 3.6.G.1.c. At approximately 1840 CST on 02/16/85, operations personnel began reactor power reduction in preparation for tripping the "B" reactor recirculation pump (1B31-C001B), because it had been determined that its shaft seals were leaking to the extent that they were causing the unacceptable quantity of drywell floor drain leakage.

At approximately 1858 CST on 02/16/85, operations personnel initiated an orderly shutdown per the "NORMAL REACTOR SHUTDOWN" procedure (HNP-1-1020), and at approximately 1859 CST on 02/16/85, operations personnel isolated and tripped the "B" reactor recirculation pump (1B31-C001B). Drywell floor drain leakage then decreased to approximately 6.0 gallons per minute by 1924 CST; hence, the 4 hour LCO for drywell floor drain leakage limits per Tech. Specs. section 3.6.G.3.a was not exceeded.

After tripping and isolating the "B" reactor recirculation pump (1B31-C001B), operations personnel reduced reactor power level to the limits of Tech. Specs. figure 3.6-5 (reactor operation with one reactor recirculation loop in operation). At approximately 2124 CST on 02/16/85, with the reactor at approximately 375 MWt, operations personnel manually scrammed the reactor. Transients following the scram proceeded as expected, and all group II isolation valves closed.

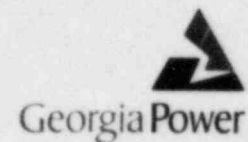
After further investigation, plant personnel determined that the number one shaft seal on the "B" reactor recirculation pump (1B31-C001B) was leaking excessively, thus causing the unacceptable volume of drywell floor drain leakage. This event is the result of material (pump shaft seal) failure.

The pump's shaft seal cartridge (which contains shaft seals number one and number two) was removed and replaced with a shaft seal cartridge which had been previously rebuilt per the manufacturer's instructions. Subsequent to repair, recirculation pump (1B31-C001B) was satisfactorily functionally tested on 02/22/85. Reactor startup then commenced on 02/22/85 at approximately 1205 CST.

No actual or potential safety consequences or implications resulted from this event; nor did it affect other systems in Unit 1, or Unit 2.

The health and safety of the public were not affected by this non-repetitive event.

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Edwin I. Hatch Nuclear Plant

March 18, 1985
GM-85-211

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-321

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1985-013. This report is required by 10CFR 50.73(a)(2)(i).



H. C. Nix
General Manager

for [unclear]
HCN/TLE/vlz

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