



**Duquesne Light Company**

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June 3, 1996  
NPD1VPO:0484

*Beaver Valley Power Station, Unit No. 1*  
*Docket No. 50-334, Licensee No. DPR-66*  
*LER-96-007-00*

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 96-007-00, 10 CFR 50.73.a.2.iv, "Inadvertent Autostart of Motor Driven Auxiliary Feed Pumps".

T. P. Noonan  
Division Vice President  
Nuclear Operations/Plant Manager

JH/jcd

Attachment

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June 3, 1996  
NPD1VPO:0484  
Page 2

cc: Mr. T. T. Martin, Regional Administrator  
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## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)

Beaver Valley Power Station Unit 1

DOCKET NUMBER (2)

05000334

PAGE (3)

1 OF 3

## Inadvertent Autostart of Motor Driven Auxiliary Feed Pumps

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	06	96	96	007	00	06	03	96	BVPS Unit 2	05000412
									FACILITY NAME	DOCKET NUMBER
										05000
OPERATING MODE (9)		3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more) (11)							
			20.402(b)			20.405(c)			X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		0	20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)	73.71(c)
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)	OTHER
			20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)	(Specify in abstract below and in Text
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)	NRC Form 366A)
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)	

## LICENSEE CONTACT FOR THIS LER (12)

NAME

T. P. Noonan, Vice President Nuclear Operations and Plant Manager

TELEPHONE NUMBER (include Area Code)

(412) 393-7622

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS				COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

## SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE)	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
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## ABSTRACT (Limited to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On May 6, 1996, both Unit 1 motor driven auxiliary feedwater (AFW) pumps inadvertently autostarted during the performance of a main feedwater pump test procedure. No significant plant transients occurred, because steam generator levels were being controlled by the feedwater bypass regulating valves and the AFW pumps were secured in a timely manner. At the time of the occurrence, Unit 1 was in Mode 3, with reactor coolant system (RCS) temperature at 360°F and pressure at 630 psig.

The cause of this event is an inadequate maintenance procedure. Corrective actions are to revise Units 1 and 2 main feedwater pump test procedures in order to incorporate expected testing conditions.

This event is reportable in accordance with 10 CFR 50.73 (a) (2) (iv), any event that resulted in an automatic actuation of an Engineered Safety Feature.

**LICENSEE EVENT REPORT (LER)**  
**TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)		DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Beaver Valley Power Station Unit 1		05000334	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
			96	007	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**PLANT AND SYSTEM IDENTIFICATION**

Westinghouse - Pressurized Water Reactor

Auxiliary Feedwater System (AF) {BA}\*

\* Energy Industry Identification System (EIIIS) codes and component function identifier codes appear in the text as {SS/CCC}.

**CONDITIONS PRIOR TO OCCURRENCE**

Unit 1: Mode 3, 0% Reactor Power

There were no structures, components, or systems that were inoperable at the start of the event that contributed to the event.

**DESCRIPTION OF EVENT:**

On May 6, 1996, Unit 1 was in Mode 3, with the RCS temperature at 360°F and pressure at 630 psig. Maintenance personnel were preparing to perform a run of the 1B main feedwater pump motors while they were uncoupled from the pump. Neither 1A nor 1B main feedwater pumps were in service; steam generator feedwater levels were being maintained by the 1A condensate pump feeding through the bypass feedwater regulating valves. In accordance with the test procedure, a jumper was installed. The purpose of the jumper was to ensure that the automatic initiation function of the motor driven auxiliary feed pumps remained operable during the test.

The 1B main feedwater pump 1B2 motor was started for its uncoupled test run. Approximately 10 seconds later, both motor driven auxiliary feedwater pumps unexpectedly autostarted. Operators then secured the 1B2 main feedwater pump motor and both motor driven auxiliary feedwater pumps.

**CAUSE OF EVENT:**

The cause of this occurrence was an inadequate test procedure. The procedure, as written, assumed that the main feed pump that was not being tested was in service. This information was not incorporated in the procedure. In the actual configuration of this event, where neither feed pump was running, moving a test pump motor control switch to the start position caused both motor driven auxiliary feedwater pumps to start.

**CORRECTIVE ACTIONS:**

1. Units 1 and 2 maintenance procedures for testing main feed pumps will be revised to ensure that uncoupled test runs will only be permitted if either one main feed pump is running or the AFW pumps are placed in the pull-to-lock (start inhibit) mode. This will be completed by August 30, 1996.

**LICENSEE EVENT REPORT (LER)**  
**TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)			PAGE (3)
Beaver Valley Power Station Unit 1		05000334		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 OF 3
				96	007	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**REPORTABILITY**

This event is reportable in accordance with 10 CFR 50.73 (a) (2) (iv), any event that resulted in an automatic actuation of an Engineered Safety Feature.

**SAFETY IMPLICATIONS:**

There were no safety implications as a result of this event. The motor driven auxiliary feedwater pumps started and performed as designed. There was no significant effect on any of the steam generator water levels as a result of this actuation because steam generator levels were being controlled by the feedwater bypass regulating valves and the AFW pumps were secured in a timely manner. Based on the above, the health and safety of the public were not affected.

**SIMILAR EVENTS**

A review of LERs for Beaver Valley Units 1 & 2 did not identify any similar occurrences within the last two years.