

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) VERMONT YANKEE NUCLEAR POWER STATION										DOCKET NUMBER (2) 0 5 0 0 4 2 7 1				PAGE (3) 1 OF 3					
TITLE (4) 1984 Appendix J-Type B and C Testing																			
EVENT DATE (6)			LER NUMBER (8)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (9)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)						
0	6	1	6	8	4	8	4	0	1	1	0	2	0	3	0	5	0	0	0
														0 5 0 0 0					
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																	
N		20.402(b)				20.406(e)				90.73(a)(2)(iv)				73.71(b)					
POWER LEVEL (10)		0 1 0 1 0				20.406(a)(1)(i)				90.73(a)(2)(v)				73.71(c)					
		20.406(a)(1)(ii)				90.73(a)(2)(vi)				90.73(a)(2)(vii)				X OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
		20.406(a)(1)(iii)				90.73(a)(2)(viii)(A)				90.73(a)(2)(viii)(B)				10CFR50 App. J.					
		20.406(a)(1)(iv)				90.73(a)(2)(ix)				90.73(a)(2)(x)									
		20.406(a)(1)(v)				90.73(a)(2)(x)													
LICENSEE CONTACT FOR THIS LER (12)																			
NAME James P. Pelletier, Plant Manager										TELEPHONE NUMBER AREA CODE 8 0 2 2 5 7 1 1									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC									
X	S	B	I I V	R	3 4 0	Y													
X	A	A	I I V	L	6 3 1	Y													
SUPPLEMENTAL REPORT EXPECTED (14)																			
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO		EXPECTED SUBMISSION DATE (15)							

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing Type C Leak Rate Testing, MSIV-86B, CRD-412A, PCAC-V16-19-8, FDW-96A and CA-89C were found to have seat leakage above that permitted by Tech. Spec. section 3.7.A.4. This resulted in the total Appendix J Type B and C limit of 14.75 lbm/Hr. being exceeded which does not meet Tech. Spec. section 3.7.A.3 requirements. (By procedure, VY uses the maximum pathway leakage in calculating total penetration leakage.)

Vermont Yankee has performed maintenance on all of the above valves and retested them to ensure that both total penetration and individual valve seat leakages are within Tech. Specs.

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PDR ADOCK 05000271  
S PDR

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
VERMONT YANKEE NUCLEAR POWER STATION	0 5 0 0 0 2 7 1 8 4	—	0 1 1	—	0 2 0	2	OF 0 3

TEXT (If more space is required, use additional NRC Form 308A (2/17))

While performing Type C Leak Rate Testing MSIV-86B, CRD-412A, PCAC V16-19-8, FDW-96A and CA-89C were found to have seat leakage above that permitted by Tech. Spec. section 3.7.A.4. This resulted in the total Appendix J Type B and C limit of 14.75 lbm/Hr being exceeded which does not meet Tech. Spec. section 3.7.A.3 requirements. (Note: By procedure, VY uses the maximum pathway leakage in calculating total penetration leakage.)

For MSIV-86B, CRD-412A, PCAC V16-19-8 and CA-89C, a second isolation valve in the applicable system was tested and met the acceptable criteria. For FDW-96A, containment was provided by the water seal on the inboard check valve and the plant's capability to maintain a pressure greater than  $P_a$  on the feed-water system.

Total penetration leakage based on the minimum pathway leakage equaled 9.83 lbm/Hr. Minimum pathway leakage is used to determine Appendix J Type A total containment leakage.

Vermont Yankee has performed maintenance on all of the above valves. For CRD-412A and CA-89C, maintenance involved disassembling the valve and cleaning the seating surface. For MSIV 86B, maintenance involved lapping the seating surface and replacing the pilot valve disc. For PCAC V16-19-8 the seats were replaced. No similar events have been reported on the above valves in the last five years. Based on this the above events are not considered significant repeat failures.

For FDW-96A, maintenance involved replacing Steven's soft seats with Stillman soft seats. The Steven's seats were scheduled to be replaced due to a known manufacturing defect. A similar event was reported on FDW 96A as LER 83-10. Vermont Yankee retested the above valves to ensure that both total penetration and individual valve seat leakages are within Tech. Specs.

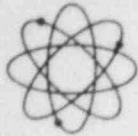
Based on the above there were no adverse consequences to the health and safety of the public.

U.S. NUCLEAR REGULATORY COMMISSION  
APPROVED ONS NO. 2150-0104  
EXPIRES: 8/31/95

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Vermont Yankee	0500027184	—	011	—	02	03	OF 03

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

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CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	
X	L E	V C	3 3 9		



# VERMONT YANKEE NUCLEAR POWER CORPORATION

P. O. BOX 157  
GOVERNOR HUNT ROAD  
VERNON, VERMONT 05354

March 8, 1985

VYV85-146

U.S. Nuclear Regulatory Commission  
Document No. 50-271  
Washington, D.C. 20555

REFERENCE: Operating License DPR-28  
Docket No. 50-271  
Reportable Occurrence No. LER 84-11, Revision 2

Dear Sirs:

As defined by 10CFR50.73, we are reporting the attached Reportable Occurrence as LER 84-11, Revision 2.

Very truly yours,

VERMONT YANKEE NUCLEAR POWER CORPORATION

James P. Pelletier  
Plant Manager

RDP/jbb

cc: Regional Administrator  
USNRC Office of Inspection and Enforcement  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

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