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May 31, 1996

U. S. Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Document Control Desk

SUBJECT: Calvert Cliffs Nuclear Power Plant
Unit Nos. 1 & 2; Docket Nos. 50-317 & 50-318
Request for Additional Information: Service Water Flow Modification to the
Containment Air Coolers

REFERENCE: (a) Letter from Mr. C. H. Cruse to NRC Document Control Desk, dated
May 28, 1996, Exigent License Amendment Request: Service Water
Flow Modification to Containment Air Coolers

As requested in a phone call from the NRC on May 29, 1996, we are providing the following information.

Question

"The intent of the valve modification is to reduce the heat transfer from the containment atmosphere to the Service Water System in order to increase the service water's capability to cool the EDGs. The Staff is concerned about the potential effect on the calculated containment peak accident pressure. Figure 14.20-1 of the Calvert Cliffs FSAR indicates that the COPATTA code containment peak pressure analysis is performed with coolant supply flow conditions to the containment air coolers of 105°F and 1900 (and 1400) gpm with a fouling factor of 0.0005. It is not clear from your May 28th letter whether the current analytical assumptions are valid for your facility. Please confirm that the most recent containment peak accident pressure analysis bounds the new operating conditions by using conservative service water supply temperature/flow rate assumptions and fouling factor assumptions for the entire pre-RAS phase of a LOCA."

Response

All assumptions in Updated Final Safety Analysis Report (UFSAR) Section 14.20 remain valid with the proposed modification. With the modification installed, the flow rate through the containment air coolers will be greater than that assumed on UFSAR Figure 14.20-1. It should be noted that the fouling factor given on UFSAR figure 14.20-1 is the fouling factor for the containment air cooler tubes, not the

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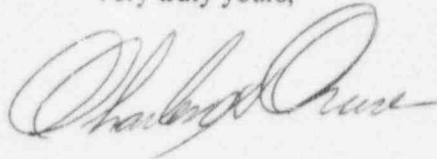
May 31, 1996

Page 2

service water heat exchanger tubes. The most recent containment peak accident pressure analysis (both pre-Recirculation Actuation Signal and post-Recirculation Actuation Signal) bounds the new operating conditions with the proposed modification installed.

This additional information does not change the Significant Hazards Determination presented in Reference (a). Should you have further questions regarding this matter, we will be pleased to discuss them with you.

Very truly yours,



CHC/PSF/dlm

cc: D. A. Byrne, Esquire
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