

ATTACHMENT "A"
(Existing Specifications)
Unit 2

- 7 -

(18) Initial Test Program (Section 14, SER)

SCE shall conduct the post-fuel loading initial test program (set forth in Section 14 of the San Onofre Units 2 and 3 Final Safety Analysis Report, as amended) without making any major modifications to this program unless such modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified in Section 14 of the Final Safety Analysis Report, as amended, as being essential.
- b. Modification of test objectives, methods, or acceptance criteria for any test identified in Section 14 of the Final Safety Analysis Report, as amended, as being essential.
- c. Performance of any test at a power level different than that described in the test procedure.
- d. Failure to complete any tests included in the described program (planned or scheduled for power levels up to the authorized power level).

(19) NUREG-0737 Conditions (Section 22)

Each of the following conditions shall be completed to the satisfaction of the NRC. Each item references the related subpart of Section 22 of the SER and/or its supplements.

a. Shift Technical Advisor (1.A.1.1, SSEK #1)

SCE shall provide a fully trained on-shift technical advisor to the shift supervisor (watch engineer).

b. Shift Manning (1.A.1.3, SSEK #1, SSEK #5)

SCE shall develop and implement administrative procedures to limit the working hours of individuals of the nuclear power plant operating staff who are responsible for manipulating plant controls or for adjusting on-line systems and equipment affecting plant safety which would have an immediate impact on public health and safety.

Adequate shift coverage shall be maintained without routine heavy use of overtime. However, in the event that unforeseen problems require substantial amounts of overtime to be used, the following guidelines shall be followed:

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1. An individual shall not be permitted to work more than 16 hours straight (excluding shift turnover time).
2. An individual shall not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven day period (all excluding shift turnover time).
3. A break of at least eight hours shall be allowed between work periods (including shift turnover time).
4. The use of overtime shall be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the station manager, his deputy, the operations manager, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime will be reviewed monthly by the station manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

c. Independent Safety Engineering Group (1.B.1.2, SSER #1)

SCE shall have an on-site independent safety engineering group.

d. Procedures for Transients and Accidents (1.C.1, SSER #1, SSER #2, SSER #3)

By May 1, 1982, SCE shall provide emergency procedure guidelines. Emergency procedures based on guidelines approved by the NRC shall be implemented prior to startup following the first fueling outage.

e. Procedures for Verifying Correct Performance of Operating Activities (1.C.6, SSER #1)

Prior to fuel loading, SCE shall implement a system for verifying the correct performance of operating activities, and shall keep the system in effect thereafter.

f. Control Room Design Review (1.D.1, SSER #1)

Prior to exceeding five (5) percent power, SCE shall:

1. Prioritize the control room annunciator windows.

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SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.1.6 -----NOTE----- Not required to be performed until 12 hours after THERMAL POWER \geq 15% RTP. ----- Verify linear power subchannel gains of the excore detectors are consistent with the values used to establish the shape annealing matrix elements in the CPCs.</p>	<p>92 days</p>
<p>SR 3.3.1.7 -----NOTES----- 1. The CPC CHANNEL FUNCTIONAL TEST shall include verification that the correct values of addressable constants are installed in each OPERABLE CPC. 2. Not required to be performed for logarithmic power level channels until 2 hours after reducing THERMAL POWER below 1E-4% RTP and only if reactor trip circuit breakers (RTCBs) are closed. ----- Perform CHANNEL FUNCTIONAL TEST on each channel except power range neutron flux.</p>	<p>92 days</p>
<p>SR 3.3.1.8 -----NOTE----- Neutron detectors are excluded from the CHANNEL CALIBRATION. ----- Perform CHANNEL CALIBRATION of the power range neutron flux channels.</p>	<p>92 days</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.8.2 -----NOTE----- This SR is applicable in MODES 1, 2, 3, and 4 only.</p> <p>-----</p> <p>Perform a CHANNEL FUNCTIONAL TEST on each required containment radiation monitor channel. Verify setpoint is in accordance with the following:</p> <p>Containment Gaseous Monitor: $\leq 2X$ background; Containment Area Gamma Monitor: ≤ 340 mR/hr.</p>	<p>92 days</p>
<p>SR 3.3.8.3 -----NOTE----- This SR is only applicable during CORE ALTERATIONS or during movement of irradiated fuel assemblies within containment.</p> <p>-----</p> <p>Perform a CHANNEL FUNCTIONAL TEST on required containment radiation monitor channel. Verify setpoint is in accordance with the following:</p> <p>Containment Gaseous Monitor: $\leq 2X$ background; Containment Area Gamma Monitor: ≤ 2.5 mR/hr.</p>	<p>92 days</p>

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.3.10.1	Perform a CHANNEL CHECK on required FHIS radiation monitor channel.	12 hours
SR 3.3.10.2	Perform a CHANNEL FUNCTIONAL TEST on required FHIS radiation monitor channel. Verify radiation monitor setpoint Allowable Values: Airborne Gaseous: $\leq 6E4$ cpm above background.	92 days
SR 3.3.10.3	-----NOTE----- Testing of Actuation Logic shall include the actuation of each initiation relay and verification of the proper operation of each ignition relay. ----- Perform a CHANNEL FUNCTIONAL TEST on required FHIS Actuation Logic channel.	18 months
SR 3.3.10.4	Perform a CHANNEL FUNCTIONAL TEST on required FHIS Manual Trip logic.	18 months
SR 3.3.10.5	Perform a CHANNEL CALIBRATION on required FHIS radiation monitor channel.	18 months

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

FUNCTION	REQUIRED CHANNELS	CONDITIONS REFERENCED FROM REQUIRED ACTION F.1
1. Excore Neutron Flux	2	G
2. Reactor Coolant System Hot Leg Temperature	2 per loop	G
3. Reactor Coolant System Cold Leg Temperature	2 per loop	G
4. Reactor Coolant System Pressure (wide range)	2	G
5. Reactor Vessel Water Level	2(d)	H
6. Containment Sump Water Level (wide range)	2	G
7. Containment Pressure (wide range)	2	G
8. Containment Isolation Valve Position	2 per penetration flow path (a)(b)	G
9. Containment Area Radiation (high range)	2	H
10. Containment Hydrogen Monitors	2	G
11. Pressurizer Level	2	G
12. Steam Generator Water Level (wide range)	2 per steam generator	G
13. Condensate Storage Tank Level	2	G
14. Core Exit Temperature - Quadrant 1	2(c)	G
15. Core Exit Temperature - Quadrant 2	2(c)	G
16. Core Exit Temperature - Quadrant 3	2(c)	G
17. Core Exit Temperature - Quadrant 4	2(c)	G
18. Auxiliary Feedwater Flow	1 per steam generator	G
19. Containment Pressure (narrow range)	2	G
20. Reactor Coolant System Subcooling Margin Monitor	2	G
21. Pressurizer Safety Valve Position Indication	1 per valve	G
22. Containment Temperature	2	G
23. Containment Water Level (narrow range)	2	G
24. HPSI Flow Cold Leg	1 per cold leg	G
25. HPSI Flow Hot Leg	1 per hot leg	G
26. Steam Line Pressure	2 per steam generator	G
27. Refueling Water Storage Tank Level	2	G

- (a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.
- (b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.
- (c) A channel consists of two or more core exit thermocouples.
- (d) A channel consists of eight sensors in a probe. A channel is OPERABLE if four or more sensors, one sensor in the upper head and three sensors in the lower head are OPERABLE.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. No SDC train/RCS loop in operation.	B.1 Suspend all operations involving reduction in RCS boron concentration.	Immediately
	<u>AND</u>	
	B.2 Initiate action to restore required SDC train/RCS loop to operation.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.4.7.1 Verify at least one RCS loop or SDC train is in operation.	12 hours
SR 3.4.7.2 Verify required SG secondary side water level is $\geq 50\%$ (wide range).	12 hours
SR 3.4.7.3 Verify the second required RCS loop, SDC train or steam generator secondary is OPERABLE.	7 days

3.9 REFUELING OPERATIONS

3.9.6 Refueling Water Level

LCO 3.9.6 Refueling water level shall be maintained ≥ 23 ft above the top of reactor vessel flange.

APPLICABILITY: During CORE ALTERATIONS, except during latching and unlatching of control rod drive shafts,
During movement of irradiated fuel assemblies within containment.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Refueling water level not within limit.	A.1 Suspend CORE ALTERATIONS.	Immediately
	<u>AND</u> A.2 Suspend movement of irradiated fuel assemblies within containment.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.9.6.1 Verify refueling water level is ≥ 23 ft above the top of reactor vessel flange.	24 hours

5.5 Procedures, Programs, and Manuals

5.5.2.12 Ventilation Filter Testing Program (VFTP) (continued)

The provisions of Technical Specification Surveillance Requirement 3.0.2 and Technical Specification Surveillance Requirement 3.0.3 are applicable to the VFTP test frequencies.

5.5.2.13 Diesel Fuel Oil Testing Program

This program implements required testing of both new fuel oil and stored fuel oil. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM standards. The purpose of the program is to establish the following:

- a. At least once per 92 days and from new fuel oil prior to addition to the storage tanks by verifying that a sample obtained in accordance with ASTM-D4057-81 has a water and sediment content of less than or equal to 0.05 volume percent, an API gravity or an absolute specific gravity within limits, and a kinematic viscosity @ 40 C of greater than or equal to 4.1 when tested in accordance with ASTM-D975-81.
 - b. At least once every 92 days by obtaining a sample of fuel oil in accordance with ASTM-D2276-81 and verifying that particulate contamination is less than 10mg/liter when checked in accordance with ASTM-D2276-83, Method A.
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ATTACHMENT "B"
(Existing Specifications)
Unit 3

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.1.6 -----NOTE----- Not required to be performed until 12 hours after THERMAL POWER \geq 15% RTP. -----</p> <p>Verify linear power subchannel gains of the excore detectors are consistent with the values used to establish the shape annealing matrix elements in the CPCs.</p>	92 days
<p>SR 3.3.1.7 -----NOTES-----</p> <ol style="list-style-type: none"> 1. The CPC CHANNEL FUNCTIONAL TEST shall include verification that the correct values of addressable constants are installed in each OPERABLE CPC. 2. Not required to be performed for logarithmic power level channels until 2 hours after reducing THERMAL POWER below 1E-4% RTP and only if reactor trip circuit breakers (RTCBs) are closed. <p>-----</p> <p>Perform CHANNEL FUNCTIONAL TEST on each channel except power range neutron flux.</p>	92 days
<p>SR 3.3.1.8 -----NOTE----- Neutron detectors are excluded from the CHANNEL CALIBRATION. -----</p> <p>Perform CHANNEL CALIBRATION of the power range neutron flux channels.</p>	92 days

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.8.2</p> <p>-----NOTE----- This SR is applicable in MODES 1, 2, 3, and 4 only. -----</p> <p>Perform a CHANNEL FUNCTIONAL TEST on each required containment radiation monitor channel. Verify setpoint is in accordance with the following:</p> <p>Containment Gaseous Monitor: $\leq 2X$ background; Containment Area Gamma Monitor: ≤ 340 mR/hr.</p>	<p>92 days</p>
<p>SR 3.3.8.3</p> <p>-----NOTE----- This SR is only applicable during CORE ALTERATIONS or during movement of irradiated fuel assemblies within containment. -----</p> <p>Perform a CHANNEL FUNCTIONAL TEST on required containment radiation monitor channel. Verify setpoint is in accordance with the following:</p> <p>Containment Gaseous Monitor: $\leq 2X$ background; Containment Area Gamma Monitor: ≤ 2.5 mR/hr.</p>	<p>92 days</p>

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.3.10.1	Perform a CHANNEL CHECK on required FHIS radiation monitor channel.	12 hours
SR 3.3.10.2	Perform a CHANNEL FUNCTIONAL TEST on required FHIS radiation monitor channel. Verify radiation monitor setpoint Allowable Values: Airborne Gaseous: $\leq 6E4$ cpm above background.	92 days
SR 3.3.10.3	-----NOTE----- Testing of Actuation Logic shall include the actuation of each initiation relay and verification of the proper operation of each ignition relay. ----- Perform a CHANNEL FUNCTIONAL TEST on required FHIS Actuation Logic channel.	18 months
SR 3.3.10.4	Perform a CHANNEL FUNCTIONAL TEST on required FHIS Manual Trip logic.	18 months
SR 3.3.10.5	Perform a CHANNEL CALIBRATION on required FHIS radiation monitor channel.	18 months

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

FUNCTION	REQUIRED CHANNELS	CONDITIONS REFERENCED FROM REQUIRED ACTION F.1
1. Excore Neutron Flux	2	G
2. Reactor Coolant System Hot Leg Temperature	2 per loop	G
3. Reactor Coolant System Cold Leg Temperature	2 per loop	G
4. Reactor Coolant System Pressure (wide range)	2	G
5. Reactor Vessel Water Level	2(d)	H
6. Containment Sump Water Level (wide range)	2	G
7. Containment Pressure (wide range)	2	G
8. Containment Isolation Valve Position	2 per penetration flow path ^{(a)(b)}	G
9. Containment Area Radiation (high range)	2	H
10. Containment Hydrogen Monitors	2	G
11. Pressurizer Level	2	G
12. Steam Generator Water Level (wide range)	2 per steam generator	G
13. Condensate Storage Tank Level	2	G
14. Core Exit Temperature - Quadrant 1	2(c)	G
15. Core Exit Temperature - Quadrant 2	2(c)	G
16. Core Exit Temperature - Quadrant 3	2(c)	G
17. Core Exit Temperature - Quadrant 4	2(c)	G
18. Auxiliary Feedwater Flow	1 per steam generator	G
19. Containment Pressure (narrow range)	2	G
20. Reactor Coolant System Subcooling Margin Monitor	2	G
21. Pressurizer Safety Valve Position Indication	1 per valve	G
22. Containment Temperature	2	G
23. Containment Water Level (narrow range)	2	G
24. HPSI Flow Cold Leg	1 per cold leg	G
25. HPSI Flow Hot Leg	1 per hot leg	G
26. Steam Line Pressure	2 per steam generator	G
27. Refueling Water Storage Tank Level	2	G

(a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) 1 channel consists of two or more core exit thermocouples.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. No SDC train/RCS loop in operation.	B.1 Suspend all operations involving reduction in RCS boron concentration.	Immediately
	<u>AND</u> B.2 Initiate action to restore required SDC train/RCS loop to operation.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.4.7.1 Verify at least one RCS loop or SDC train is in operation.	12 hours
SR 3.4.7.2 Verify required SG secondary side water level is $\geq 50\%$ (wide range).	12 hours
SR 3.4.7.3 Verify the second required RCS loop, SDC train or steam generator secondary is OPERABLE.	7 days

3.9 REFUELING OPERATIONS

3.9.6 Refueling Water Level

LC0 3.9.6 Refueling water level shall be maintained ≥ 23 ft above the top of reactor vessel flange.

APPLICABILITY: During CORE ALTERATIONS, except during latching and unlatching of control rod drive shafts,
During movement of irradiated fuel assemblies within containment.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Refueling water level not within limit.	A.1 Suspend CORE ALTERATIONS.	Immediately
	<u>AND</u> A.2 Suspend movement of irradiated fuel assemblies within containment.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.9.6.1 Verify refueling water level is ≥ 23 ft above the top of reactor vessel flange.	24 hours

5.5 Procedures, Programs, and Manuals

5.5.2.12 Ventilation Filter Testing Program (VFTP) (continued)

The provisions of Technical Specification Surveillance Requirement 3.0.2 and Technical Specification Surveillance Requirement 3.0.3 are applicable to the VFTP test frequencies.

5.5.2.13 Diesel Fuel Oil Testing Program

This program implements required testing of both new fuel oil and stored fuel oil. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM standards. The purpose of the program is to establish the following:

- a. At least once per 92 days and from new fuel oil prior to addition to the storage tanks by verifying that a sample obtained in accordance with ASTM-D4057-81 has a water and sediment content of less than or equal to 0.05 volume percent, an API gravity or an absolute specific gravity within limits, and a kinematic viscosity @ 40 C of greater than or equal to 4.1 when tested in accordance with ASTM-D975-81.
 - b. At least once every 92 days by obtaining a sample of fuel oil in accordance with ASTM-D2276-81 and verifying that particulate contamination is less than 10mg/liter when checked in accordance with ASTM-D2276-83, Method A.
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ATTACHMENT "C"
(Proposed Specifications)
Unit 2

- 7 -

(18) Initial Test Program (Section 14, SER)

SCE shall conduct the post-fuel loading initial test program (set forth in Section 14 of the San Onofre Units 2 and 3 Final Safety Analysis Report, as amended) without making any major modifications to this program unless such modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified in Section 14 of the Final Safety Analysis Report, as amended, as being essential.
- b. Modification of test objectives, methods, or acceptance criteria for any test identified in Section 14 of the Final Safety Analysis Report, as amended, as being essential.
- c. Performance of any test at a power level different than that described in the test procedure.
- d. Failure to complete any tests included in the described program (planned or scheduled for power levels up to the authorized power level).

(19) NUREG-0737 Conditions (Section 22)

Each of the following conditions shall be completed to the satisfaction of the NRC. Each item references the related subpart of Section 22 of the SER and/or its supplements.

a. Shift Technical Advisor (I.A.1.1, SSER #1)

SCE shall provide a fully trained on-shift technical advisor to the shift supervisor (watch engineer).

Deleted.

b. Shift Manning (I.A.1.3, SSER #1, SSER #5)

SCE shall develop and implement administrative procedures to limit the working hours of individuals of the nuclear power plant operating staff who are responsible for manipulating plant controls or for adjusting on-line systems and equipment affecting plant safety which would have an immediate impact on public health and safety.

Adequate shift coverage shall be maintained without routine heavy use of overtime. However, in the event that unforeseen problems require substantial amounts of overtime to be used, the following guidelines shall be followed:

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1. An individual shall not be permitted to work more than 16 hours straight (excluding shift turnover time).
2. An individual shall not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any seven day period (all excluding shift turnover time).
3. A break of at least eight hours shall be allowed between work periods (including shift turnover time).
4. The use of overtime shall be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the station manager, his deputy, the operations manager, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime will be reviewed monthly by the station manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

c. Independent Safety Engineering Group (I.B.1.2, SSER #1)

SCE shall have an on-site independent safety engineering group.

d. Procedures for Transients and Accidents (I.C.1, SSER #1, SSER #2, SSER #3)

By May 1, 1982, SCE shall provide emergency procedure guidelines. Emergency procedures based on guidelines approved by the NRC shall be implemented prior to startup following the first refueling outage.

e. Procedures for Verifying Correct Performance of Operating Activities (I.C.6, SSER #1)

Prior to fuel loading, SCE shall implement a system for verifying the correct performance of operating activities, and shall keep the system in effect thereafter.

f. Control Room Design Review (I.D.1, SSER #1)

Prior to exceeding five (5) percent power, SCE shall:

1. Prioritize the control room annunciator windows.

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SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.1.6 -----NOTE----- Not required to be performed until 12 hours after THERMAL POWER \geq 15% RTP. ----- Verify linear power subchannel gains of the excore detectors are consistent with the values used to establish the shape annealing matrix elements in the CPCs.</p>	92 days
<p>SR 3.3.1.7 -----NOTES----- 1. The CPC CHANNEL FUNCTIONAL TEST shall include verification that the correct values of addressable constants are installed in each OPERABLE CPC. 2. Not required to be performed for logarithmic power level channels until 2 hours after reducing THERMAL POWER below 1E-4% RTP and only if reactor trip circuit breakers (RTCBs) are closed. ----- Perform CHANNEL FUNCTIONAL TEST on each channel except power range neutron flux</p>	92 days
<p>SR 3.3.1.8 -----NOTE----- Neutron detectors are excluded from the CHANNEL CALIBRATION. ----- Perform CHANNEL CALIBRATION of the power range neutron flux channels.</p>	92 days

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.8.2 -----NOTE----- This SR is applicable in MODES 1, 2, 3, and 4 only.</p> <p>Perform a CHANNEL FUNCTIONAL TEST on each required containment radiation monitor channel. Verify setpoint is in accordance with the following:</p> <p>Containment Gaseous Monitor: ≤ 2X background Containment Area Gamma Monitor: ≤ 340 mR/hr.</p>	<p>92 days</p>
<p>SR 3.3.8.3 -----NOTE----- This SR is only applicable during CORE ALTERATIONS or during movement of irradiated fuel assemblies within containment.</p> <p>Perform a CHANNEL FUNCTIONAL TEST on required containment radiation monitor channel. Verify setpoint is in accordance with the following:</p> <p>Containment Gaseous Monitor: ≤ 2X background Containment Area Gamma Monitor: ≤ 2.5 mR/hr.</p>	<p>92 days</p>

(continued)

The trip setpoint shall be set sufficiently high to prevent spurious alarms/trips yet sufficiently low to ensure an alarm/trip should an inadvertent release occur.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.3.10.1 Perform a CHANNEL CHECK on required FHIS radiation monitor channel.	12 hours
SR 3.3.10.2 Perform a CHANNEL FUNCTIONAL TEST on required FHIS radiation monitor channel. Verify radiation monitor setpoint Allowable Values. The trip setpoint shall be set sufficiently high to prevent spurious alarms/trips yet Airborne Gaseous. $\leq 6E4$ cpm above background <i>sufficiently low to ensure an alarm/trip should a fuel handling accident occur.</i>	92 days
SR 3.3.10.3 -----NOTE----- Testing of Actuation Logic shall include the actuation of each initiation relay and verification of the proper operation of each ignition relay. ----- Perform a CHANNEL FUNCTIONAL TEST on required FHIS Actuation Logic channel.	18 months
SR 3.3.10.4 Perform a CHANNEL FUNCTIONAL TEST on required FHIS Manual Trip logic.	18 months
SR 3.3.10.5 Perform a CHANNEL CALIBRATION on required FHIS radiation monitor channel.	18 months

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

FUNCTION	REQUIRED CHANNELS	CONDITIONS REFERENCED FROM REQUIRED ACTION F.1
1. Excore Neutron Flux	2	G
2. Reactor Coolant System Hot Leg Temperature	2 (per loop)	G
3. Reactor Coolant System Cold Leg Temperature	2 (per loop)	G
4. Reactor Coolant System Pressure (wide range)	2	G
5. Reactor Vessel Water Level	2(d)	H
6. Containment Sump Water Level (wide range)	2	G
7. Containment Pressure (wide range)	2	G
8. Containment Isolation Valve Position	2 per penetration flow path(a)(b)	G
9. Containment Area Radiation (high range)	2	H
10. Containment Hydrogen Monitors	2	G
11. Pressurizer Level	2	G
12. Steam Generator Water Level (wide range)	2 per steam generator	G
13. Condensate Storage Tank Level	2	G
14. Core Exit Temperature - Quadrant 1	2(c)	G
15. Core Exit Temperature - Quadrant 2	2(c)	G
16. Core Exit Temperature - Quadrant 3	2(c)	G
17. Core Exit Temperature - Quadrant 4	2(c)	G
18. Auxiliary Feedwater Flow	1 per steam generator	G
19. Containment Pressure (narrow range)	2	G
20. Reactor Coolant System Subcooling Margin Monitor	2	G
21. Pressurizer Safety Valve Position Indication	1 per valve	G
22. Containment Temperature	2	G
23. Containment Water Level (narrow range)	2	G
24. HPSI Flow Cold Leg	1 per cold leg	G
25. HPSI Flow Hot Leg	1 per hot leg	G
26. Steam Line Pressure	2 per steam generator	G
27. Refueling Water Storage Tank Level	2	G

- (a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.
- (b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.
- (c) A channel consists of two or more core exit thermocouples.
- (d) A channel consists of eight sensors in a probe. A channel is OPERABLE if four or more sensors, one sensor in the upper head and three sensors in the lower head are OPERABLE.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. No SDC train/RCS loop in operation.	B.1 Suspend all operations involving reduction in RCS boron concentration.	Immediately
	<u>AND</u>	
	B.2 Initiate action to restore required SDC train/RCS loop to operation.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.4.7.1 Verify at least one RCS loop or SDC train is in operation.	12 hours
SR 3.4.7.2 Verify required SG secondary side water level is \geq 50% (wide range). ?	12 hours
SR 3.4.7.3 Verify the second required RCS loop, SDC train or steam generator secondary is OPERABLE.	7 days

3.9 REFUELING OPERATIONS

3.9.6 Refueling Water Level

LCO 3.9.6 Refueling water level shall be maintained ≥ 23 ft above the top of reactor vessel flange.

NOTE: Water level may be lowered to a minimum of 23 feet above the top of the fuel for movement of four finger CEAs, coupling and uncoupling of CEA extension shafts or for verifying the coupling and uncoupling.

APPLICABILITY: ~~During CORE ALTERATIONS, except during latching and unlatching of control rod drive shafts,~~
During movement of irradiated fuel assemblies within containment.
During movement of fuel assemblies or CEAs within the reactor pressure vessel when either the fuel assemblies being moved or the fuel assemblies seated within the reactor pressure vessel are irradiated.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Refueling water level not within limit.	A.1 Suspend CORE ALTERATIONS.	Immediately
	AND A.2 Suspend movement of irradiated fuel assemblies within containment.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.9.6.1 <i>The</i> Verify refueling water level is ≥ 23 ft above the top of reactor vessel flange. <i>shall be determined to be at least its minimum required depth.</i>	24 hours

5.5 Procedures, Programs, and Manuals

5.5.2.12 Ventilation Filter Testing Program (VFTP) (continued)

The provisions of Technical Specification Surveillance Requirement 3.0.2 and Technical Specification Surveillance Requirement 3.0.3 are applicable to the VFTP test frequencies.

5.5.2.13 Diesel Fuel Oil Testing Program

This program implements required testing of both new fuel oil and stored fuel oil. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM standards. The purpose of the program is to establish the following:

- a. ~~At least once per 92 days and~~ from new fuel oil prior to addition to the storage tanks by verifying that a sample obtained in accordance with ASTM-D4057-81 has a water and sediment content of less than or equal to 0.05 volume percent, an API gravity or an absolute specific gravity within limits, and a kinematic viscosity @ 40 C of greater than or equal to 4.1 when tested in accordance with ASTM-D975-81.
1.9 but less than or equal to D4057-81
 - b. At least once every 92 days by obtaining a sample of fuel oil in accordance with ASTM-D2276-81 and verifying that particulate contamination is less than 10mg/liter when checked in accordance with ASTM-D2276-83, Method A.
-

ATTACHMENT "D"
(Proposed Specifications)
Unit 3

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.1.6 -----NOTE----- Not required to be performed until 12 hours after THERMAL POWER \geq 15% RTP. ----- Verify linear power subchannel gains of the excore detectors are consistent with the values used to establish the shape annealing matrix elements in the CPCs.</p>	92 days
<p>SR 3.3.1.7 -----NOTES----- 1. The CPC CHANNEL FUNCTIONAL TEST shall include verification that the correct values of addressable constants are installed in each OPERABLE CPC. 2. Not required to be performed for logarithmic power level channels until 2 hours after reducing THERMAL POWER below 1E-4% RTP and only if reactor trip circuit breakers (RTCBs) are closed. ----- Perform CHANNEL FUNCTIONAL TEST on each channel except power range neutron flux</p>	- 92 days
<p>SR 3.3.1.8 -----NOTE----- Neutron detectors are excluded from the CHANNEL CALIBRATION. ----- Perform CHANNEL CALIBRATION of the power range neutron flux channels.</p>	92 days

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.8.2</p> <p>-----NOTE----- This SR is applicable in MODES 1, 2, 3, and 4 only.</p> <p>-----</p> <p>Perform a CHANNEL FUNCTIONAL TEST on each required containment radiation monitor channel. Verify setpoint is in accordance with the following:</p> <p>Containment Gaseous Monitor: ≤ 2X background Containment Area Gamma Monitor: ≤ 340 mR/hr.</p>	92 days
<p>SR 3.3.8.3</p> <p>-----NOTE----- This SR is only applicable during CORE ALTERATIONS or during movement of irradiated fuel assemblies within containment.</p> <p>-----</p> <p>Perform a CHANNEL FUNCTIONAL TEST on required containment radiation monitor channel. Verify setpoint is in accordance with the following:</p> <p>Containment Gaseous Monitor: ≤ 2X background Containment Area Gamma Monitor: ≤ 2.5 mR/hr.</p>	92 days
	(continued)

The trip setpoint shall be set sufficiently
high to prevent spurious alarms/trips yet
sufficiently low to ensure an alarm/trip
should an inadvertent release occur.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.3.10.1 Perform a CHANNEL CHECK on required FHIS radiation monitor channel.	12 hours
SR 3.3.10.2 Perform a CHANNEL FUNCTIONAL TEST on required FHIS radiation monitor channel. Verify radiation monitor setpoint Allowable values. <i>The trip setpoint shall be set sufficiently high to prevent spurious alarms/trips yet sufficiently low to ensure an alarm/trip should a fuel handling accident occur.</i> Airborne Gaseous: $\leq 5E4$ cpm above background.	92 days
SR 3.3.10.3 -----NOTE----- Testing of Actuation Logic shall include the actuation of each initiation relay and verification of the proper operation of each ignition relay. ----- Perform a CHANNEL FUNCTIONAL TEST on required FHIS Actuation Logic channel.	18 months
SR 3.3.10.4 Perform a CHANNEL FUNCTIONAL TEST on required FHIS Manual Trip logic.	18 months
SR 3.3.10.5 Perform a CHANNEL CALIBRATION on required FHIS radiation monitor channel.	18 months

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

FUNCTION	REQUIRED CHANNELS	CONDITIONS REFERENCED FROM REQUIRED ACTION F.1
1. Excore Neutron Flux	2	G
2. Reactor Coolant System Hot Leg Temperature	2 per loop	G
3. Reactor Coolant System Cold Leg Temperature	2 per loop	G
4. Reactor Coolant System Pressure (wide range)	2	G
5. Reactor Vessel Water Level	2(d)	H
6. Containment Sump Water Level (wide range)	2	G
7. Containment Pressure (wide range)	2	G
8. Containment Isolation Valve Position	2 per penetration flow path (a)(b)	G
9. Containment Area Radiation (high range)	2	H
10. Containment Hydrogen Monitors	2	G
11. Pressurizer Level	2	G
12. Steam Generator Water Level (wide range)	2 per steam generator	G
13. Condensate Storage Tank Level	2	G
14. Core Exit Temperature - Quadrant 1	2(c)	G
15. Core Exit Temperature - Quadrant 2	2(c)	G
16. Core Exit Temperature - Quadrant 3	2(c)	G
17. Core Exit Temperature - Quadrant 4	2(c)	G
18. Auxiliary Feedwater Flow	1 per steam generator	G
19. Containment Pressure (narrow range)	2	G
20. Reactor Coolant System Subcooling Margin Monitor	2	G
21. Pressurizer Safety Valve Position Indication	1 per valve	G
22. Containment Temperature	2	G
23. Containment Water Level (narrow range)	2	G
24. HPSI Flow Cold Leg	1 per cold leg	G
25. HPSI Flow Hot Leg	1 per hot leg	G
26. Steam Line Pressure	2 per steam generator	G
27. Refueling Water Storage Tank Level	2	G

(a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) A channel consists of two or more core exit thermocouples.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. No SDC train/RCS loop in operation.	B.1 Suspend all operations involving reduction in RCS boron concentration.	Immediately
	<u>AND</u> B.2 Initiate action to restore required SDC train/RCS loop to operation.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.4.7.1 Verify at least one RCS loop or SDC train is in operation.	12 hours
SR 3.4.7.2 Verify required SG secondary side water level is 50 50% (wide range). >	12 hours
SR 3.4.7.3 Verify the second required RCS loop, SDC train or steam generator secondary is OPERABLE.	7 days

3.9 REFUELING OPERATIONS

3.9.6 Refueling Water Level

LCO 3.9.6 Refueling water level shall be maintained ≥ 23 ft above the top of reactor vessel flange.

NOTE: Water level may be lowered to a minimum of 23 feet above the top of the fuel for movement of four finger CEAs, coupling and uncoupling of CEA extension shafts or for verifying the coupling and uncoupling.

APPLICABILITY: *During CORE ALTERATIONS, except during latching and unlatching of control rod drive shafts, During movement of irradiated fuel assemblies within containment. During movement of fuel assemblies or CEAs within the reactor pressure vessel when either the fuel assemblies being moved or the fuel assemblies seated within the reactor pressure vessel are irradiated.*

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Refueling water level not within limit.	A.1 Suspend CORE ALTERATIONS.	Immediately
	<u>AND</u> A.2 Suspend movement of irradiated fuel assemblies within containment.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.9.6.1 <i>The</i> Verify refueling water level is ≥ 23 fte above the top of reactor vessel flange. <i>shall be determined to be at least its minimum required depth.</i>	24 hours

5.5 Procedures, Programs, and Manuals

5.5.2.12 Ventilation Filter Testing Program (VFTP) (continued)

The provisions of Technical Specification Surveillance Requirement 3.0.2 and Technical Specification Surveillance Requirement 3.0.3 are applicable to the VFTP test frequencies.

5.5.2.13 Diesel Fuel Oil Testing Program

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- b. At least once every 92 days by obtaining a sample of fuel oil in accordance with ASTM-D2276-81 and verifying that particulate contamination is less than 10mg/liter when checked in accordance with ASTM-D2276-83, Method A.