



Northeast
Nuclear Energy

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The Northeast Utilities System

MAY 24 1996

Docket No. 50-336
B15724

Re: 10 CFR 50.73

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

This letter forwards Licensee Event Report (LER) 96-023-00 documenting an event that occurred at Millstone Nuclear Power Station, Unit No. 2 on April 25, 1996. This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(i).

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

P. M. Richardson
Director - Millstone Unit No. 2

Attachment: LER 96-023-00

cc: T. T. Martin, Region I Administrator
P. D. Swetland, Senior Resident Inspector, Millstone Unit No. 2
D. G. McDonald, Jr., NRC Project Manager, Millstone Unit No. 2

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LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY
INFORMATION COLLECTION REQUEST: 50.9 HRS. REPORTED LESSONS
LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED
BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN
ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-
6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC
20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104),
OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Millstone Nuclear Power Station Unit 2

DOCKET NUMBER (2)

05000336

PAGE (3)

1 of 3

TITLE (4)

Failure to Perform Technical Specifications Surveillances on Certain Containment Isolation Valves

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	25	96	96	023	00	05	24	96	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)							
POWER LEVEL (10)		0%	20.2201(b)			20.2203(a)(2)(v)			<input checked="" type="checkbox"/> 50.73(a)(2)(i)	50.73(a)(2)(viii)
			20.2203(a)(1)			20.2203(a)(3)(i)			50.73(a)(2)(ii)	50.73(a)(2)(x)
			20.2203(a)(2)(i)			20.2203(a)(3)(ii)			50.73(a)(2)(iii)	73.71
			20.2203(a)(2)(ii)			20.2203(a)(4)			50.73(a)(2)(iv)	OTHER
			20.2203(a)(2)(iii)			50.36(c)(1)			50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A
			20.2203(a)(2)(iv)			50.36(c)(2)			50.73(a)(2)(vii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME

G. P. van Noordennen, Nuclear Licensing Supervisor

TELEPHONE NUMBER (Include Area Code)

(860)440-2084

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION	MONTH	DAY	YEAR
(If yes, complete EXPECTED SUBMISSION DATE).					

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 25, 1996 at 1520 hours, with the plant in mode 5 at 0% power, an internal audit discovered that several valves located within containment isolation boundaries were not being inspected to verify they were in the closed position. This monthly check demonstrates containment integrity and is required to be performed at least once per 31 days, in accordance with the requirements of Technical Specifications (TS) section 4.6.1.1.a. This event is being reported pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications."

The cause of this event was an inadequate procedure that did not include several valves that must be verified as closed to maintain containment integrity. The failure of these valves to be included on the checklist is the result of an error made during the development or revision of the procedures.

The isolation valves that had not been inspected in accordance with the TS were subsequently inspected and verified to be in the closed position. Procedure changes will be made to add the missing valves to the appropriate forms.

There were no automatic or manually initiated safety systems actuated as result of this event.

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Millstone Nuclear Power Station Unit 2	05000336					2 of 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

I. Description of Event

On April 25, 1996 at 1520 hours, with the plant in mode 5 at 0% power, an internal audit discovered that several valves located within containment isolation boundaries were not being inspected to verify they were in the closed position. This monthly check demonstrates containment integrity and is required to be performed at least once per 31 days, in accordance with the requirements of Technical Specifications (TS) section 4.6.1.1.a.

There were no automatic or manually initiated safety systems actuated as result of this event. Additionally no operator action was required in response to this event.

II. Cause of Event

The cause of this event was an inadequate procedure that did not include several valves that must be verified as closed to maintain containment integrity. The failure of these valves to be included on the checklist is the result of an error made during the development or revision of the procedures.

III. Analysis of Event

This event is being reported pursuant to the requirements of 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications." The investigation concluded that this event was reportable since valves required to maintain containment integrity were not verified to be closed in accordance with the requirements of TS section 4.6.1.1.a.

The procedural discrepancies identified in this event were discovered by an internal audit team tasked with reviewing TS compliance. The Final Safety Analysis Report (FSAR) table 5.2-11 lists containment structure isolation valve information. The audit found that some valves listed in the FSAR table, were not listed on the monthly surveillance procedure (OPS Form 2605A-1) and, therefore, operators were not verifying the valves were in the closed position.

The valves being investigated, to determine if they should be included in the surveillance procedure, are 3/4 inch and smaller vent and drain valves, a 2 inch isolation valve (2-CH-517) located on the pressurizer auxiliary spray line, and two 8 inch main steam line atmospheric dump valves (MS-190 A & B). The 31 day surveillance was not performed on these valves, however, an inspection was performed to verify these valves are closed. Additionally, the investigation determined that all but two 3/8" drain valves were previously inspected as required by other surveillance procedures, however, not within a 31 day cycle.

The actual and potential safety significance of this event is low since subsequent to the event the valves were verified to be closed.

IV. Corrective Action

The isolation valves that had not been inspected in accordance with the TS were subsequently inspected and verified to be in the closed position.

Procedure changes will be made to add valves, found to require the 30 day surveillance, to the appropriate forms including OPS Form 2605A-1. Additionally, other TS surveillances, containing requirements to verify valve

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

position, are being reviewed to identify any valves that are potentially not included within appropriate surveillance procedures.

V. Additional Information

Similar Events

None

Manufacturer Data

None