



Northern States Power Company
Prairie Island Nuclear Generating Plant
1717 Wakonade Dr. East
Welch, Minnesota 55089

May 20, 1996

Bulletin 96-01

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Summary Report for Prairie Island Rod Drop Data (requested
by Bulletin 96-01 from Unit 2 Reactor Trip of April 18, 1996

This letter is to summarize results of Rod Drop testing performed at Prairie Island Nuclear Generating Plant following a Unit 2 Reactor Trip which occurred on April 18, 1996. This testing was performed per requested action #3 of NRC Bulletin 96-01 Control Rod Insertion Problems.

Testing was performed identically to the normal refueling test by simultaneously dropping all rods from a full out position at hot (547°F) full pressure (2235 psig) full flow (100%) condition. The results of this test were essentially identical to testing previously reported on April 18, 1996 (within ± 30 msec). Of the 29 rods tested the highest exposure rod was approximately 42,938 MWD/MTU. All of the rods tested met the technical specification drop time requirements. No abnormalities were detected in rod drop time (rod full out to entry to dash pot region) nor in total rod drop time (full out to rod bottom). All rods experienced recoil upon rod bottom similar to normal refueling rod drop testing. In comparison of rod drop times to data submitted April 18, 1996, no degradation in drop time nor rod recoil was evident.

The testing performed verified operability of each of the control rods and did not require any corrective actions.

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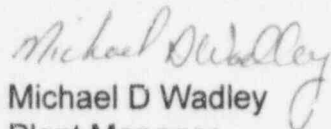
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NORTHERN STATES POWER COMPANY

In addition attached to this letter is a corrected Unit 2 burnup table from our April 8, 1996 original response letter showing the correct "current" date of 2/29/96 rather than 2/29/95 in the column header.

In this letter we have made no new Nuclear Regulatory Commission commitments.

Please contact Jack Leveille (612-388-1121, Ext. 4662) if you have any questions related to this letter.



Michael D Wadley
Plant Manager
Prairie Island Nuclear Generating Plant

c: Regional Administrator - Region III, NRC
Senior Resident Inspector, NRC
NRR Project Manager, NRC
J E Silberg

Attachments:

1. Affidavit
2. Fuel Burnup Data for Unit Two Cycle Seventeen

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

DOCKET NO. 50-282
50-306

BULLETIN 96-01, CONTROL ROD INSERTION PROBLEMS

Northern States Power Company, a Minnesota corporation, with this letter is submitting information requested by NRC Bulletin 96-01.

This letter contains no restricted or other defense information.

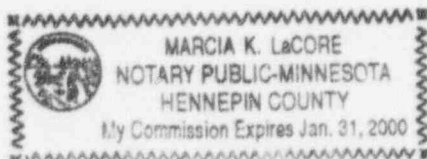
NORTHERN STATES POWER COMPANY

BY

Michael D Wadley
Michael D Wadley
Plant Manager
Prairie Island Nuclear Generating Plant

On this 20th day of May 1996 before me a notary public in and for said County, personally appeared Michael D Wadley, Plant Manager, Prairie Island Nuclear Generating Plant; and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof, and that to the best of his knowledge, information, and belief the statements made in it are true and that it is not interposed for delay.

Marcia K. LaCore



Unit Two Cycle Seventeen

Fuel Assembly	Core Location	Control Rod	Control Rod Bank Group	Burnup At BOC Timing Test (MWD/MTU)	Burnup on 2/29/96 (MWD/MTU)	Projected EOC Exposure (MWD/MTU)
P-22	G07	R-148	C2	35775	41776	50490
T-51	D04	R-132	C2	23749	34301	47340
T-53	D10	R-147	C1	24097	34762	47340
T-54	J10	R-138	C2	24222	34899	47340
T-56	J04	R-141	C1	24020	34788	47340
T-66	K07	R-158	D1	24381	35656	49020
T-67	C07	R-152	D1	24453	35677	49020
T-68	G03	R-151	D1	24387	35642	49020
T-69	G11	R-150	D1	25120	36395	49020
T-82	F08	R-145	B1	27669	37392	50100
T-83	F06	R-157	B1	27895	37828	50100
T-84	H06	R-134	B1	27960	37763	50100
T-85	H08	R-142	B1	27611	37372	50100
U-66	L08	R-131	A2	0	11243	25720
U-67	F12	R-154	A2	0	11501	25720
U-68	B05	R-136	A2	0	11091	25720
U-69	H02	R-139	A2	0	11013	25720
U-70	H12	R-143	A1	0	11443	25720
U-71	B08	R-135	A1	0	10943	25720
U-72	F02	R-156	A1	0	11084	25720
U-73	L03	R-153	A1	0	11292	25720
U-90	K09	R-144	S.B1	0	11829	28070
U-91	E11	R-137	S.B1	0	11924	28070
U-92	C05	R-140	S.B1	0	11788	28070
U-93	I03	R-133	S.B1	0	11820	28070
U-94	I11	R-155	S.A1	0	11991	28070
U-95	C09	R-146	S.A2	0	11876	28070
U-96	E03	R-149	S.A1	0	11655	28070
U-97	K05	R-159	S.A2	0	11828	28070

Fuel Burnup Data