



Commonwealth Edison

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March 8, 1985

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Byron Generating Station Units 1 and 2
Braidwood Generating Station Units 1 and 2
FSAR Changes
NRC Docket Nos. 50-454/455 and 50-456/457

Dear Mr. Denton:

The letter provides advance copies of revised pages for the Byron/Braidwood FSAR. These pages are being revised to incorporate the results of revised reactor transient analyses. These pages will be incorporated into the FSAR at the earliest opportunity. NRC review is required.

Attachment A to this letter lists the revised FSAR pages which are enclosed. These pages document the Westinghouse reanalysis of the reactor's transient response to a complete loss of forced reactor coolant flow. The reanalysis demonstrates that the minimum DNBR will remain above 1.3, considering the as-built reactor coolant pump performance.

The characteristic of the reactor coolant flow decay is an important input to this transient analysis. The startup test described in FSAR Table 14.2-70 is conducted to verify that the FSAR analysis is conservative with respect to flow coastdown. Startup testing has determined that adequate margin exists for Byron 1. Future testing is expected to confirm the margin for the other three units.

Use of the as-built pump data resulted in a revision to a startup test acceptance criterion after the test review was completed. At the direction of I&E Region III personnel, this change was reviewed as a change to the Post-Fuel-Loading Initial Test Program in accordance with Condition 3 of the Byron 1 Operating Licensing NPF-37. A report of this change will be submitted to Region III in accordance with 50.59(b) and Condition 3.

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H. R. Denton

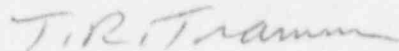
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Please address further questions this matter to this office.

One signed original and fifteen copies of this letter and the enclosure are provided for NRC review.

Very truly yours,



T. R. Tramm
Nuclear Licensing Administrator

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cc: J. G. Keppler - Region III
Byron Resident Inspector

Attachment

9860N

ATTACHMENT A

LIST OF REVISED FSAR PAGES

Page 15.3-15, Table 15.3-1	time sequence of events
Figure 15.3-9	flow vs. time
Figure 15.3-10(a)	power vs. time
Figure 15.3-10(b)	pressure vs. time
Figure 15.3-11(a)	heat flux vs. time (aug. channel)
Figure 15.3-11(b)	heat flux vs. time (hot channel)
Figure 15.3-12	DNBR vs. time

TABLE 15.3-1 (Cont'd)

TIME SEQUENCE OF EVENTS FOR INCIDENTS
WHICH RESULT IN A DECREASE IN REACTOR COOLANT
SYSTEM FLOW

<u>ACCIDENT</u>	<u>EVENT</u>	<u>TIME (sec)</u>	
		<u>Four Loop Operation</u>	<u>Three Loop Operation</u>
	Reactor coolant pump under-voltage trip point reached	0.0 ✓	0.0
	Rods begin to drop	1.5 ✓	1.5
	Minimum DNBR occurs	3.7 3.4	3.5
Reactor Coolant Pump Shaft Seizure (Locked Rotor)	Rotor on one pump locks	0.0	0.0
	Low flow trip point reached	0.04	0.03
	Rods begin to drop	1.0	1.03

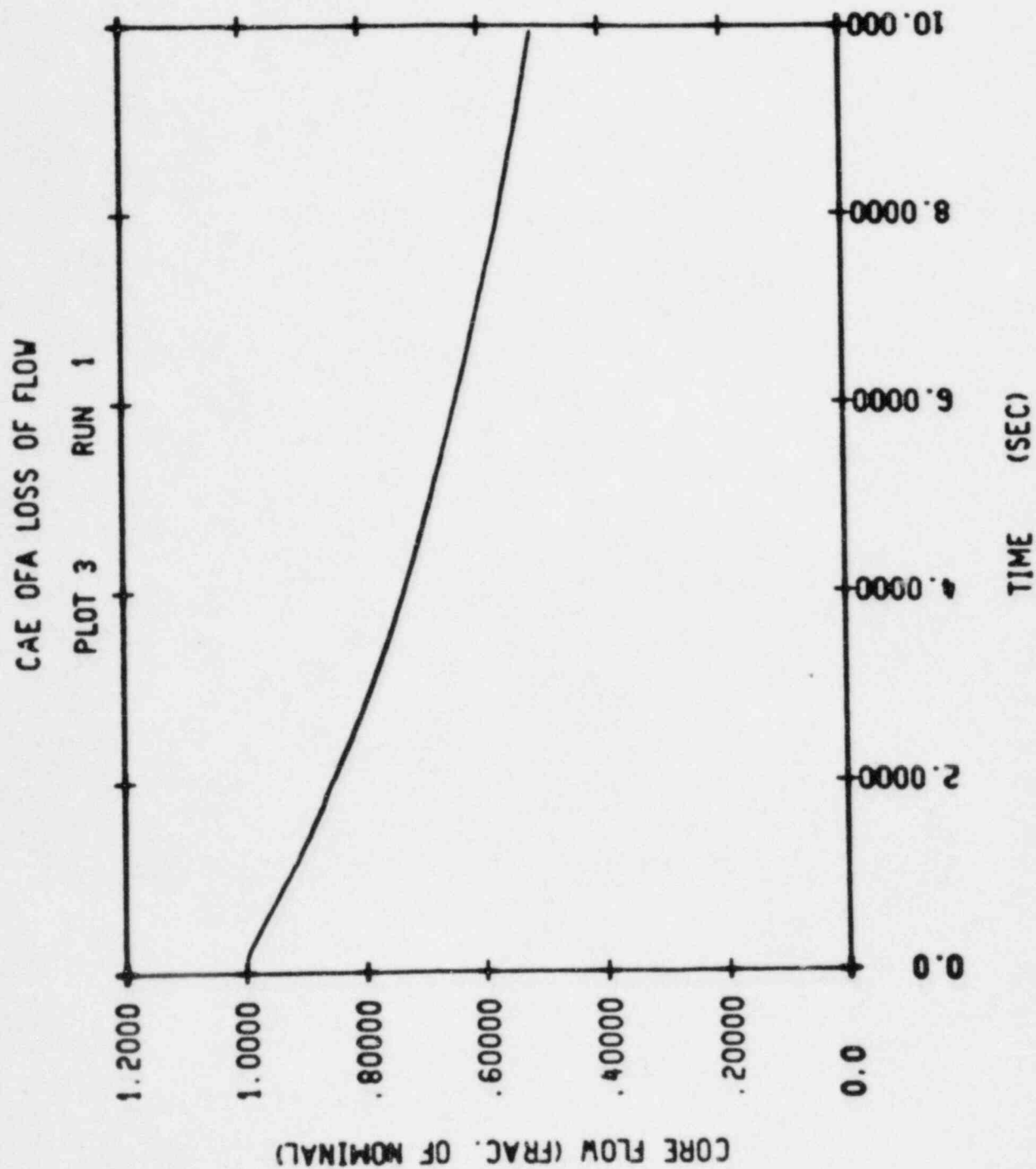


FIGURE 15.3-9

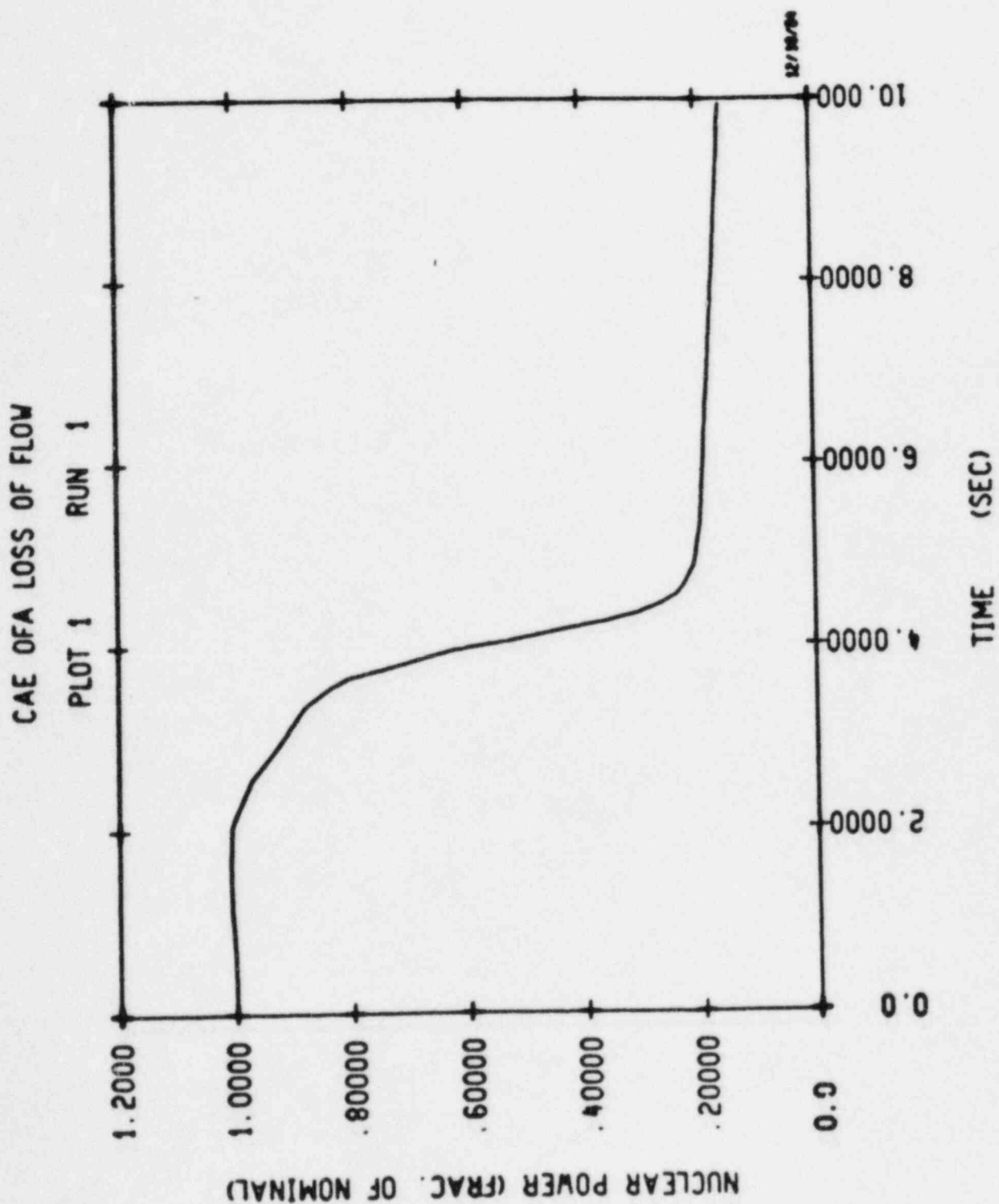


FIGURE 15.3-10

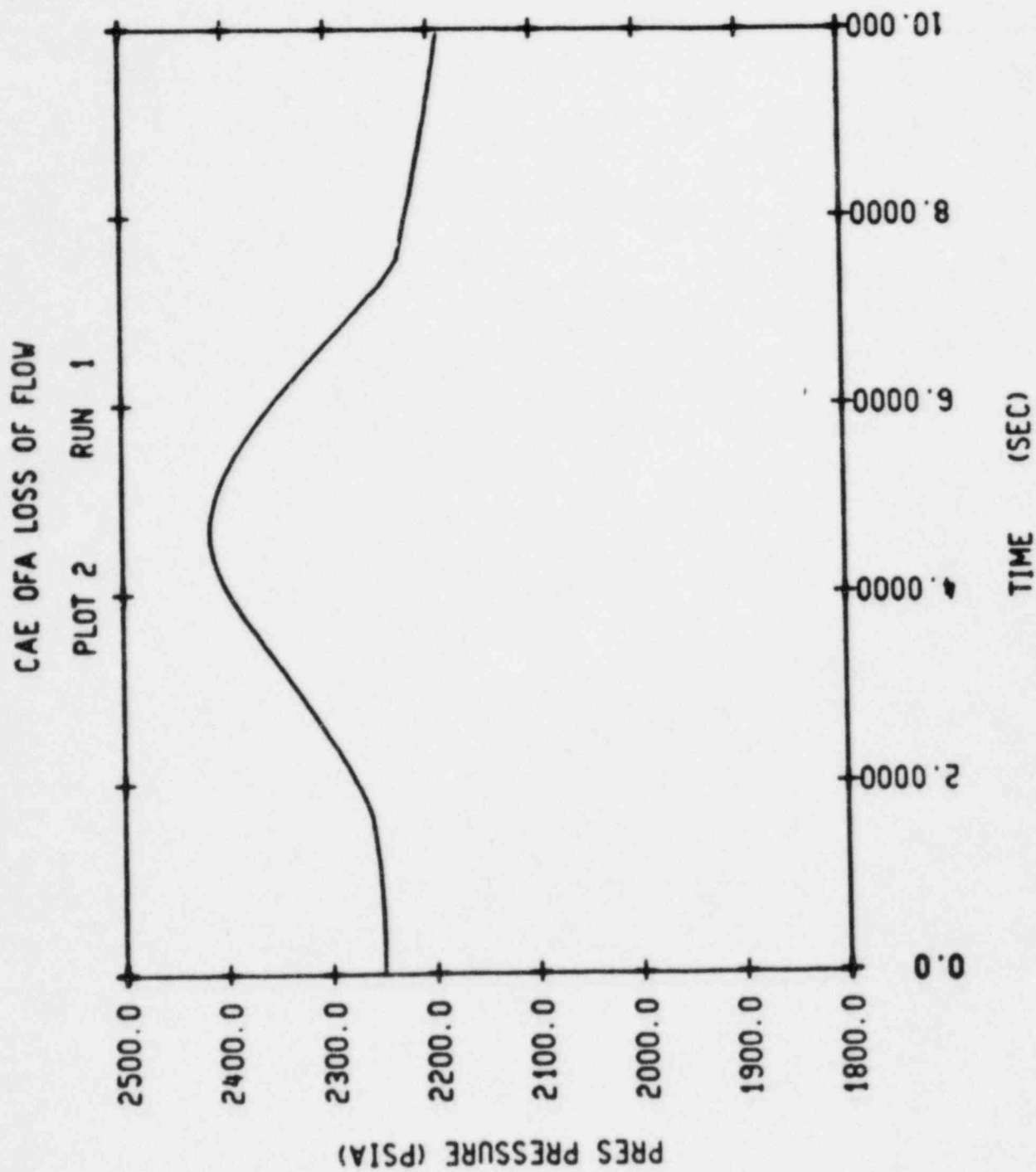


FIGURE 15.3-10(b)

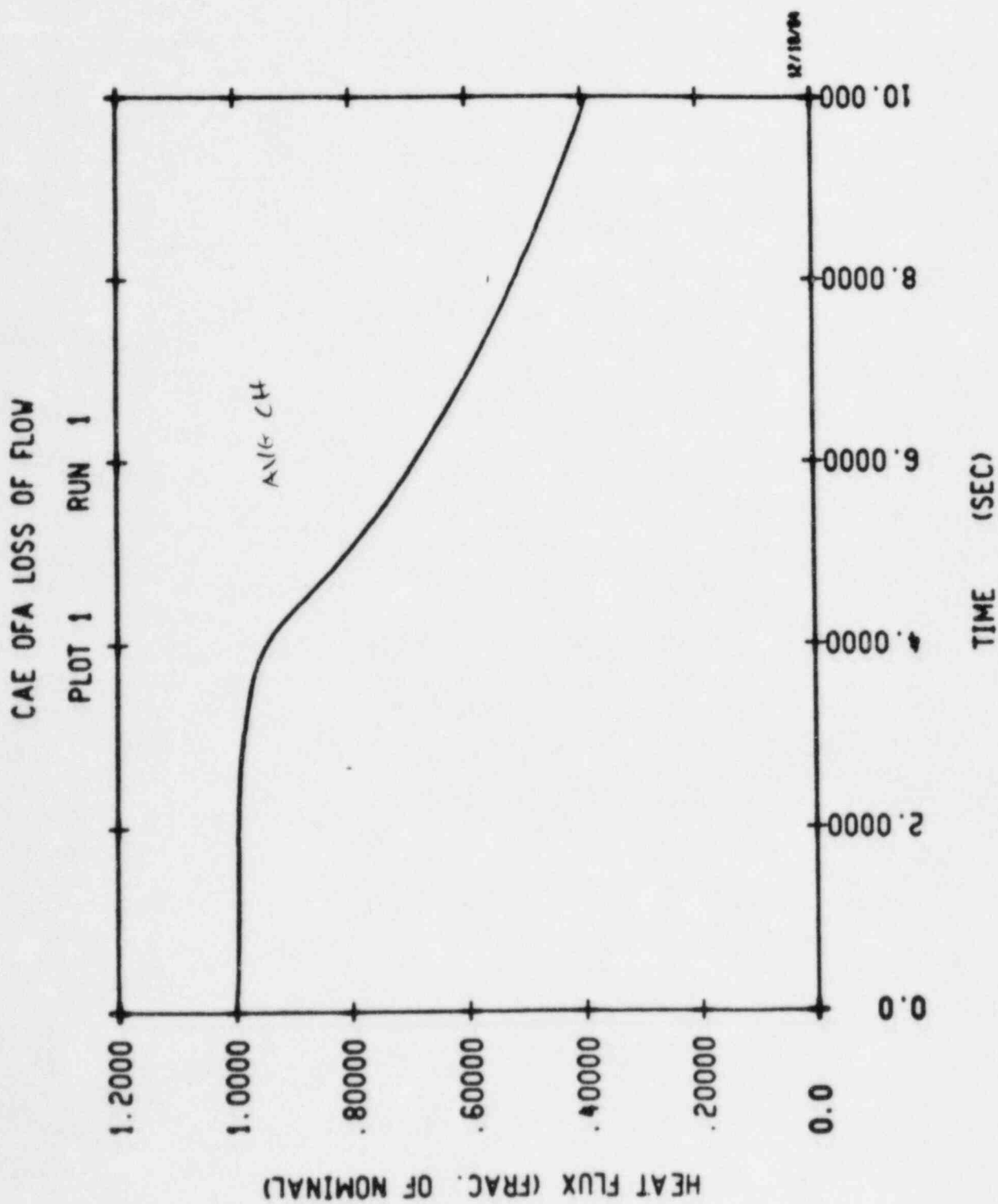


FIGURE 15.3-11(a)

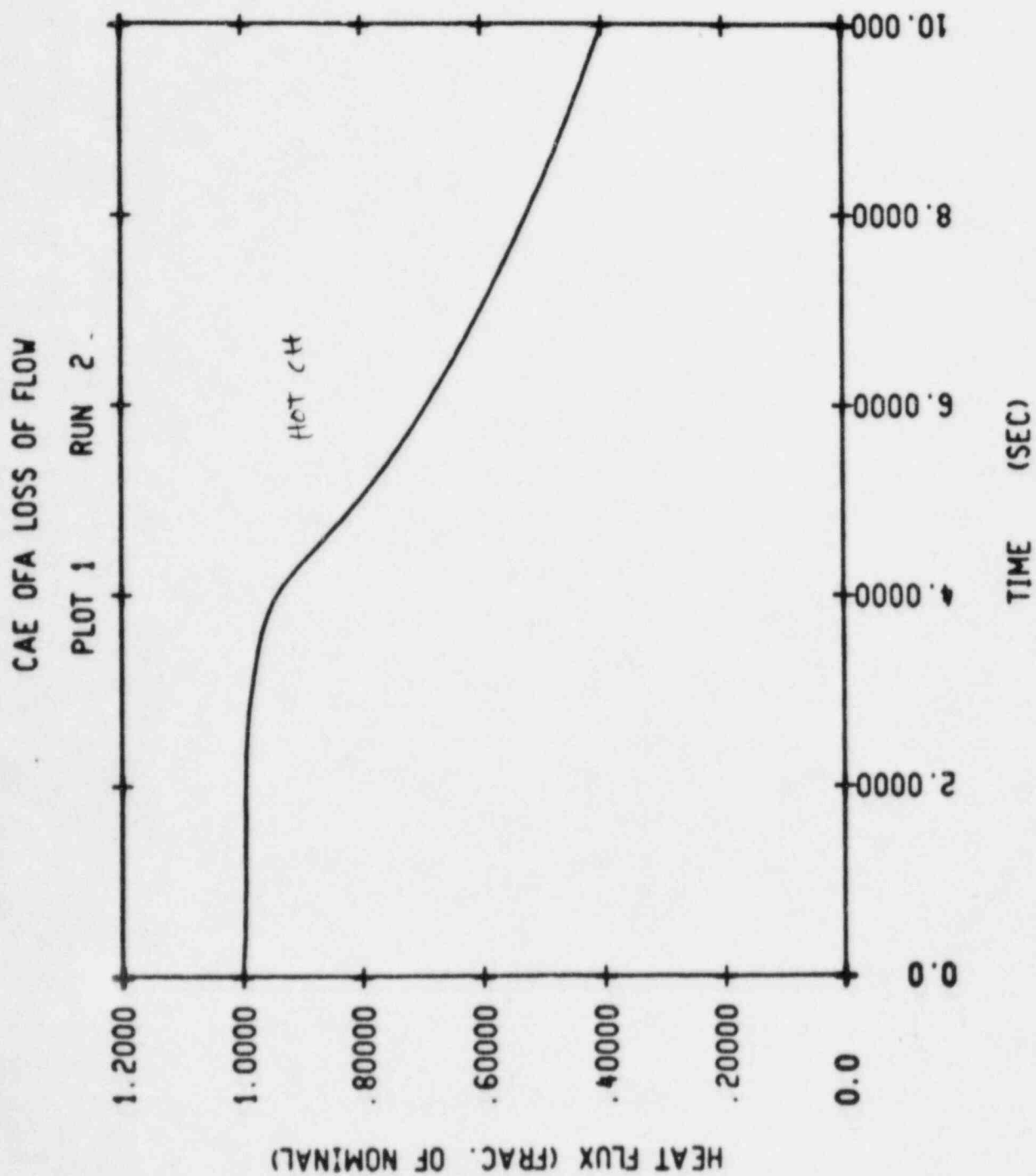


FIGURE 15.3 - 11(b)

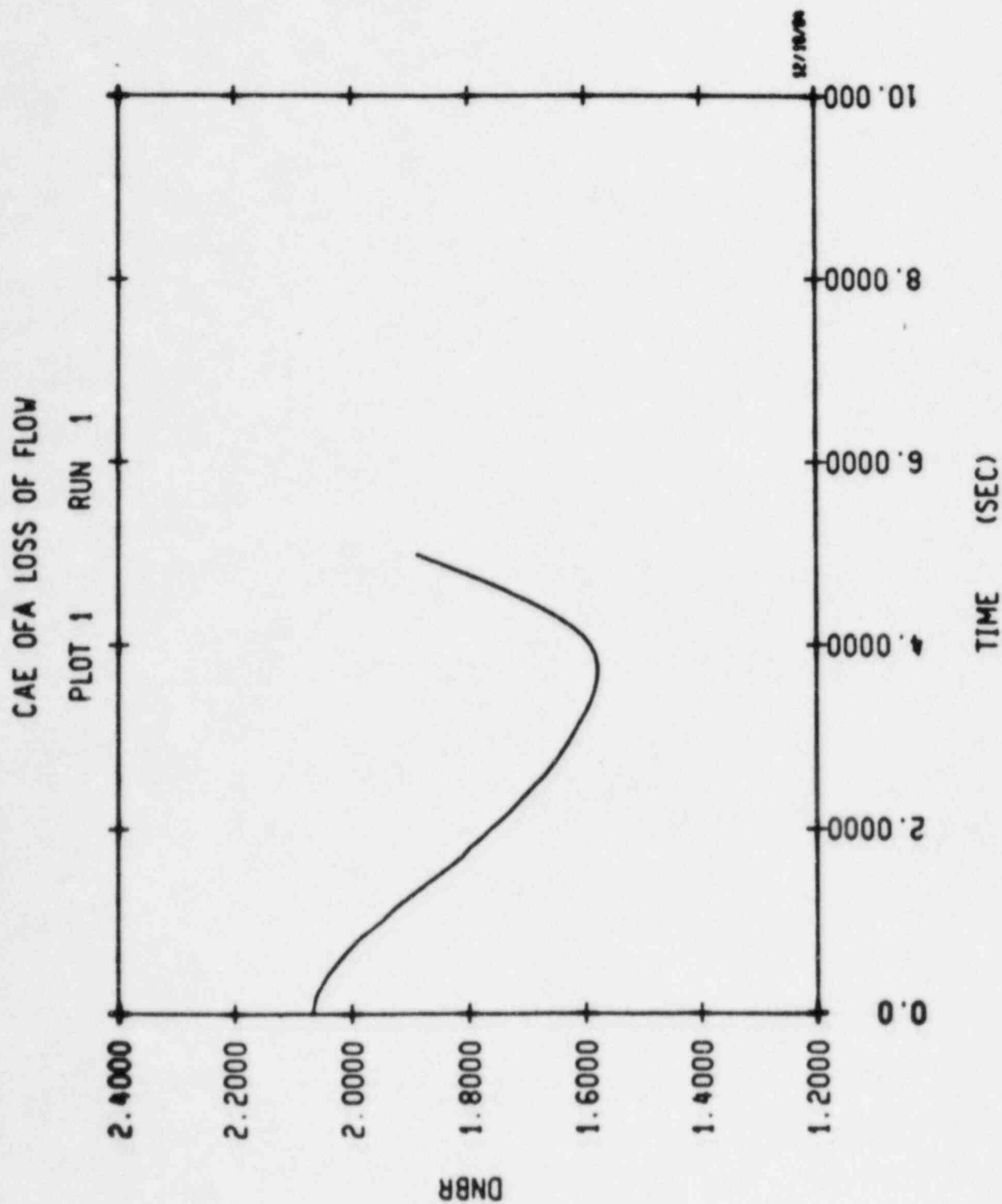


FIGURE 15.3-12