

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Waterford 3 Steam Electric Station

DOCKET NUMBER (2)

0 5 0 0 0 3 8 2 1 OF 0 3

PAGE (3)

TITLE (4)

Inadvertent Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)										
0	2	0	9	8	5	8	5	0	0	4	0	0	0	3	8	2	1	OF	0	3	
										N/A	0	5	0	0	0						
										N/A	0	5	0	0	0						

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)									
POWER LEVEL (10)	01010	20.402(b)	20.406(c)	X	50.73(a)(2)(iv)	73.71(b)					
		20.406(a)(1)(i)	50.38(c)(1)		50.73(a)(2)(v)	73.71(c)					
		20.406(a)(1)(ii)	50.38(c)(2)		50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)					
		20.406(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(vii)(A)						
		20.406(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(vii)(B)						
		20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)						TELEPHONE NUMBER					
NAME						AREA CODE					
O.D. Hayes, Operations Superintendent						51014 46141-1311118					

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

On February 9, 1985 Waterford 3 Steam Electric Station was in mode 3 when an inadvertent reactor trip occurred while Plant Personnel were installing heat shrink on the Logarithmic Power Level Nuclear Instrumentation.

This event was reported to the Commission pursuant to 10CFR50.72 (b)(2)(ii).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1): Waterford 3 Steam Electric Station	DOCKET NUMBER (2): 0 5 0 0 0 3 8 2 8 5 - 0 0 4 - 0 0	LER NUMBER (6)			PAGE (3):		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 305A's) (17)

NARRATIVE

On February 9, 1985 the Waterford 3 Steam Electric Station was in mode 3 when Instruments and Controls Technicians were installing heat shrink on the Logarithmic Power Level Nuclear Instrumentation wiring in channel A of the Plant Protection System. Plant conditions were established in accordance with Phase III Test Procedure SIT-TP-503, Control Element Drive Mechanism Performance Testing. In addition to the above conditions, Plant Protection System channel B LOW DNBR and HIGH LOCAL POWER DENSITY bistables were in the trip condition, and HIGH LINEAR POWER LEVEL and HIGH LOGARITHMIC POWER LEVEL bistables were in bypass in order to install a choke in the Excore Nuclear Instrumentation. Channel C LOW DNBR and HIGH LOCAL POWER DENSITY bistables were in the bypass condition due to Resistance Temperature Detector response time testing as described in procedure MI-3-209.

At 1108 CST an uncomplicated reactor trip (a trip in which primary system parameters are within normal range as defined in Attachment 1 of OP-902-000, Emergency Entry Procedure) occurred due to a LOW DNBR and HIGH LOCAL POWER DENSITY trip in channel A of the Plant Protection System satisfying the two out of four actuation logic (channels A and B). When the trip occurred Operations Personnel performed Emergency Operating Procedures OP-902-000, Emergency Entry Procedure, and OP-902-001, Uncomplicated Reactor Trip Recovery. As required by Operating Procedure OP-10-001, General Plant Operations, the Shift Technical Advisor performed a Post-Trip Review and determined the event to be a Type I trip as defined in Attachment 8.19 of the subject procedure.

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an uncomplicated reactor trip during Phase III Testing in which no primary system parameters were exceeded. Since the Control Element Assemblies and the Reactor Protection System functioned as designed, the subject event in no way placed Waterford 3 in a degraded safety condition.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTION

In an effort to prevent such events from occurring in the future, the Waterford 3 Maintenance Superintendent has developed a list of guidelines for all Maintenance Personnel when performing work on high risk equipment. These guidelines are outlined in Maintenance Directive number 12, Program to Minimize Inadvertent ESF Actuations and Plant Trips. In addition, the Operations Superintendent will issue an Operations Daily Instruction to inform Operations Personnel of general requirements when work is being conducted on safety-related equipment.

SIMILAR EVENTS

None

PLANT CONTACT

O.D. Hayes, Operations Superintendent, 504/464-3118



MIDDLE SOUTH
UTILITIES SYSTEM

LOUISIANA
POWER & LIGHT

142 DELARONDE STREET
NEW ORLEANS, LOUISIANA

P.O. BOX 8008
70174-8008

(504) 386-2345

March 8, 1985

W3P85-0608
3-A1.01.04
C14.03
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-26
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-004-00 for the Waterford 3
Steam Electric Station. This Licensee Event Report is submitted per
10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook

Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), E.L. Blake,
W.M. Stevenson

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