GPU Nuclear

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May 15,1996

6730-96-2164

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Dear Sir:

SUBJECT: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report - April 1996

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1, enclosed are two (2) copies of the Monthly Operating Data (Gray Book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Ms. Brenda DeMerchant, Oyster Creek Regulatory Affairs Engineer, at 609-971-4642.

Very truly yours,

Michael B Roche

Michael B. Roche
Vice President & Director
Oyster Creek

MBR/BDeM/gl

Enclosures

cc: Administrator, Region I (2 copies)
NRC Project Manager
NRC Resident Inspector

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# SUMMARY
# APRIL, 1996

Oyster Creek operated at full load during the first week of April with minor power reductions for Control Rod Drive and Hydraulic Control Unit testing. On April 9, a Technical Specification shutdown commenced after Emergency Bus Grid under voltage relays were declared inoperable due to incomplete testing. Operations terminated the shutdown at 85% power after testing was completed. Full power was again achieved at 0200 on April 10. At 2053 on April 10, operators reduced power to 75% in response to a drop in "C" Condenser's vacuum during backwash. The vacuum drop was a result of reduced circulating water flow through the "C" Condenser. The reduced flow occurred when faulty contacts prevented the proper positioning of valves used to backwash the condensers. The valves were repositioned and repaired, and the plant returned to full load early on April 11.

On April 18, Operations commenced a planned power reduction to perform Main Steam Isolation Valve testing. During the power reduction, the "A" Feedwater Pump seals were replaced and radiography was performed on the Condensate flow orifice. Radiography identified that a broken Steam Packing Exhauster Condenser flapper valve was lodged at the flow orifice. Reactor power was administratively restricted to 70% to minimize condensate flow transients. Full shutdown to recover the flapper valve occurred on April 26. In addition to recovering the flapper, during the shutdown the large 1-1 Auxiliary Flash Tank Pump, and the Steam Seal Header drain line (an air leak source into the "A" Condenser) were replaced.

Plant start-up commenced on April 29. On April 30, the reactor tripped on low water level at 10% power. Plant startup commenced again later on April 30, with the generator going on line on May 1.

During the month, the plant generated 350,506 MWh net electric which was 75.7% of its MDC rated capacity.

## LICENSEE EVENT REPORTS FILED DURING APRIL 1996

96-001 On 03/13/96 maintenance work was performed on the Secondary Containment Isolation Valves V-28-0042 & 0043. The post-maintenance test to stroke these valves one at a time could not be performed using the existing procedure. Further review revealed that these valves had not been functionally tested to demonstrate full tech spec compliance since 1987.

The root cause of the missed surveillances on V-28-0042 & 0043 was a narrow focus on the part of the preparers & reviewers of the modification in that the modification package concentrated on the electrical aspects of the modification & did not ensure the basic intent of the modification would be tested & maintained. This resulted in an inadequate surveillance procedure.

Upon discovery of the condition, the two valves were tested to verify automatic closure in response to an isolation signal. Procedures relating to secondary containment isolation valves will be reviewed & revised as necessary to include testing of V-28-0042 & 0043. A modification to the RB ductwork will be made to allow leak testing for these two valves. Additionally, the procedures controlling the modification process will be reviewed to determine if any additional revisions would be beneficial.

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96-002 During a recent review of the battery charger test and calibration procedure it was discovered that the procedural low voltage alarm setpoint (120± 1 vdc) was outside the Technical Specification requirement (115± 1 vdc). The root cause of this occurrence has been determined to be an improper work practice in that the new test and calibration procedure had been written without an adequate review of technical specification requirements. The safety significance has been determined to be minimal as the incorrect setpoint was more conservative than the required setpoint and would have alarmed sooner than required by design.

Upon discovery, the battery charger test and calibration procedure was immediately revised and the relay setpoint was adjusted to comply with the technical specification requirements. To prevent a similar occurrence, the technical specifications related to this occurrence will be reviewed to determine if they can be clarified by an amendment. Additionally, personnel involved with the review and approval of procedures will be informed of this event and advised to ensure that technical specification limits are considered even when the proposed change increases the margin of safety.

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET # . . . . . 50-219
UNIT . . . . . Oyster Creek #1
REPORT DATE . . . . . 5/6/96
COMPILED BY. . . . . Paul G. Edelmann
TELEPHONE #. . . . . (609) 971- 4097

Month: April, 1996

| DAY | MW | DAY | MW |
|---|---|---|---|
| 1 | 631 | 16 | 636 |
| 2 | 631 | 17 | 637 |
| 3 | 634 | 18 | 611 |
| 4 | 638 | 19 | 377 |
| 5 | 626 | 20 | 395 |
| 6 | 623 | 21 | 394 |
| 7 | 627 | 22 | 393 |
| 8 | 637 | 23 | 393 |
| 9 | 625 | 24 | 396 |
| 10 | 615 | 25 | 379 |
| 11 | 635 | 26 | 2 |
| 12 | 638 | 27 | 0 |
| 13 | 637 | 28 | 0 |
| 14 | 637 | 29 | 0 |
| 15 | 636 | 30 | 0 |

Oyster Creek Station #1

Docket No. 50-219

REFUELING INFORMATION - APRIL, 1996

| | |
|---|---|
| Name of Facility: | Oyster Creek Station #1 |
| Scheduled date for next refueling shutdown: | September, 1996 |
| Scheduled date for restart following refueling: | Currently projected for November, 1996 |

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment? No

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - Fuel design and performance analysis methods have been approved by the NRC.

| | | | |
|---|---|---|---|
| The number of fuel assemblies | (a) in the core | = | 560 |
| | (b) in the spent fuel storage pool | = | 2048 |
| | (c) in dry storage | = | 24 |

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present Licensed Capacity: 2645

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core discharge capacity to the spent fuel pool will be lost after the 1996 refueling outage.

# OPERATING DATA REPORT

## OPERATING STATUS

| | |
|---|---|
| 1. Docket: | 50-219 |
| 2. Reporting Period: | April 1996 |
| 3. Utility Contact | Paul G. Edelmann (609)-971-4097 |
| 4. Licensed Thermal Power (MWt): | 1930 |
| 5. Nameplate Rating (Gross MWe): | 687.5 x 0.8 = 550 |
| 6. Design Electrical Rating (Net MWe) | 650 |
| 7. Maximum Dependable Capacity (Gross MWe): | 641 |
| 8. Maximum Dependable Capacity (Net MWe): | 619 |
| 9. If Changes Occur Above Since Last Report, Give Reasons: | None |
| 10. Power Level to Which Restricted, If Any (Net MWe): | None |
| 11. Reason For Restriction, If Any: | None |

| | Month | Year | Cumulative |
|---|---|---|---|
| 12. Report Period Hours | 719.0 | 2903.0 | 231023.0 |
| 13. Hours RX Critical | 619.5 | 2767.6 | 157100.5 |
| 14. RX Reserve Shutdown Hours | 0.0 | 0.0 | 918.2 |
| 15. Hours Generator On-Line | 600.2 | 2726.1 | 153536.7 |
| 16. UT Reserve Shutdown Hours | 0.0 | 0.0 | 0.0 |
| 17. Gross Thermal Energy (MWH) | 1049007 | 5057213 | 265576706 |
| 18. Gross Electric Energy (MWH | 350506 | 1723892 | 89141467 |
| 19. Net Electric Energy (MWH | 336672 | 1660992 | 85530157 |
| 20. UT Service Factor | 83.5 | 93.9 | 66.5 |
| 21. UT Available Factor | 83.5 | 93.9 | 66.5 |
| 22. UT Capacity Factor (MDC Net) | 75.6 | 92.4 | 60.4 |
| 23. UT Capacity Factor (DER Net) | 72.0 | 88.0 | 57.0 |
| 24. UT Forced Outage Rate | 16.5 | 6.1 | 9.8 |
| 25. Forced Outage Hours | 118.9 | 177.0 | 16715.2 |

26. Shutdowns Scheduled Over Next 6 Months (Type, Date, Duration)

    16R Outage, September 7, 1996; 45 Days

27. If Currently Shutdown, Estimated Startup Date: N/A

# UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-219
UNIT NAME: Oyster Creek
DATE: May 07, 1996
COMPLT'D BY: David M. Egan
TELEPHONE: 609/971-4818

REPORT MONTH: April 1996

| No. | DATE | TYPE F: Forced S: Scheduled | DURATION (hours) | REASON (1) | METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2) | CORRECTIVE ACTIONS/COMMENTS |
|---|---|---|---|---|---|---|
| 3 | 04/18/96 | S | 0 | b | 1 | Operators manually reduced power to 35% to perform MSIV full closure surveillance and other maintenance activities. |
| 4 | 04/26/96 | F | 118.9 | b/g | 1/3 | Operators manually shutdown the reactor to retrieve a broken Steam Packing Exhauster Condenser flapper valve lodged in front of the Condensate flow orifice.<br><br>An automatic scram on low reactor water level occurred during the subsequent startup. The root cause of this event has been determined to be a failure of the CRO to aggressively control reactor water level in the control band during the start-up using station operating procedures. The scram occurred prior to the turbine going on line (10% reactor power). |

SUMMARY:

(1) REASON
a. Equipment Failure (Explain)
b. Maintenance or Test
c. Refueling
d. Regulatory Restriction
e. Operator Training & Lic Exam
f. Administrative
g. Operational Error (Explain)
h. Other (Explain)

(2) METHOD
1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)