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May 15, 1996
NPD1VPO:0476

Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, Licensee No. DPR-66
LER-96-006-00

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 96-006-00, Unit 1, Inadequate Testing of Safety Injection Relays, is reported in accordance with 10 CFR 50.73a.2.i.B, (Condition Prohibited by Technical Specifications).

This report is being submitted beyond the 30 day reporting requirements of 10 CFR 50.73 due to a noncompliance with the administrative controls for the evaluation of reportable plant events and conditions.

200042

T. P. Noonan
Division Vice President
Nuclear Operations/Plant Manager

GFZ/jcd

Attachment

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cc: Mr. T. T. Martin, Regional Administrator
United States Nuclear Regulatory Commission
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King of Prussia, PA 19406

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Virginia Electric & Power Company
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Richmond, VA 23261

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Beaver Valley Power Station Unit 1

DOCKET NUMBER (2)

05000334

PAGE (3)

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Inadequate Testing of Safety Injection Relays

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	15	96	96	006	00	05	15	96	N/A	05000
FACILITY NAME										
DOCKET NUMBER										
05000										

OPERATING MODE (9)	6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more) (11)								
POWER LEVEL (10)	0	20.402(b)			20.405(c)			50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)		OTHER
		20.405(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(viii)(A)		(Specify in abstract below and in Text NRC Form 366A)
		20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)		
20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)				

LICENSEE CONTACT FOR THIS LER (12)

NAME

T. P. Noonan, Vice President Nuclear Operations and Plant Manager

TELEPHONE NUMBER (include Area Code)

(412) 393-7622

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS				COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
D	BP	XXX	XXXX	N						

SUPPLEMENTAL REPORT EXPECTED (14)

YES	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
(if yes, complete EXPECTED SUBMISSION DATE)						

ABSTRACT (Limited to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On 4/15/96, while in mode 6, defueled conditions, during a review of a Safety System Functional Evaluation (SSFE) self-assessment, it was determined that failure to test the Safety Injection Automatic Transfer to Recirculation latch-in relays, constituted a condition not permitted by the Technical Specifications. The SSFE self-assessment had identified the failure to test the relays in November 1995, and Operating Surveillance Tests were revised to include a correct test methodology. These revisions were completed on 12/1/95, and satisfactory testing of the two trains of equipment was performed on 12/28/95 and 12/29/95. A review of the Unit 2 test procedures was conducted and were found to be adequate.

In response to an NRC inspector evaluation of the self-assessment, a reportability and operability re-evaluation was performed. The re-evaluation conducted on 4/15/96, concluded that testing of the latch-in relays is required to comply with Technical Specification Surveillance Requirement 4.5.2.f.2 and as such this condition is prohibited by Technical Specifications and reportable in accordance with 10CFR50.73.a.2.i.B. The operability and reportability assessment done in 1995, was not performed in accordance with administrative procedures which provides the instructions in conducting operability and reportability evaluations of plant events. As a result, the time frame to submit this licensee event report was exceeded.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Beaver Valley Power Station Unit 1	05000334	96	006	00	2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A (17))

DESCRIPTION OF EVENT

A Safety System Functional Evaluation (SSFE) self-assessment, completed in November 1995, noted that latch-in relays were employed in the Solid State Protection System for the Safety Injection Transfer to Recirculation logic. It was observed that although the latch-in relays are designed to maintain the Safety Injection trip signal after the Safety Injection signal has been reset, the functional testing then being performed did not ensure that this signal was maintained. This self-assessment recommended that the Operating Surveillance Tests that are performed on these relays be revised to properly test the latch-in function. Another item identified in this self-assessment was the need to evaluate the operability and reportability of the inadequate testing of the latch-in relays.

The revisions to the Operating Surveillance Tests were completed on 12/1/95, and testing that properly verified the latch-in relay function of both trains of Safety Injection Automatic Transfer to Recirculation was completed on 12/29/95. The re-evaluation of operability and reportability, which was conducted in response to an NRC evaluation of the self-assessment, determined that testing performed previous to the surveillance test revisions did not satisfy the Technical Specification Surveillance Requirement 4.5.2.f.2, and was reportable under 10CFR50.73.a.2.i.B, as a condition prohibited by Technical Specifications.

CAUSE OF EVENT

The cause of the failure to functionally test the Safety Injection Automatic Transfer to Recirculation latch-in relays (inadequate surveillance), did not identify the need to include the latch-in relays as part of the Technical Specifications surveillance. The cause of the late submittal of this licensee event report was due to the inadequate implementation of administrative controls for evaluating plant events.

CORRECTIVE ACTIONS

The following corrective actions have been taken as a result of this event:

1. Operating Surveillance Tests were revised to test the seal-in feature of the relays involved.
2. The Safety Injection Automatic Transfer to Recirculation Latch was tested satisfactorily.
3. An evaluation for the Unit 2 latch-In relays was conducted and concluded that the Operating Surveillance Tests are adequate.
4. The individuals involved in the initial operability and reportability determination were counseled on the proper methods described in administrative procedures to conduct operability and reportability evaluations.

REPORTABILITY

The latching function of the Safety Injection Automatic Transfer to Recirculation relay was not tested as required by Technical Specification Surveillance Requirement 4.5.2.f.2. Accordingly this report is being submitted in accordance with 10CFR50.73.a.2.i.B, as an event or condition prohibited by Technical Specifications. The submittal beyond the 30 days required by 10CFR 50.73 is explained above.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

SAFETY IMPLICATIONS

There were no safety implications due to this event. These relays were tested satisfactorily following the Operating Surveillance Test revision. The malfunction of the late-in feature of these relays does not prevent the Safety Injection System from performing its design basis function.

SIMILAR EVENTS

In the past two years Beaver Valley Power Station Unit 1 has submitted one LER due to inadequate surveillance.

1-95-009 "ASME Valves not Tested Within Technical Specification Surveillance Interval".

There have been no recent LERS due to late reporting.