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May 13, 1996
6710-96-2166

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for April 1996

Enclosed are two copies of the April 1996 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

J. Knubel
Vice President and Director, TMI

9605200407 960430
PDR ADOCK 05000289
R PDR

WGH

Attachment

cc: Administrator, Region I
TMI Senior Resident Inspector
96001

OPERATIONS SUMMARY

April 1996

The plant entered the month operating at 100% power and remained at that power level for the remainder of the month. Net unit electrical output averaged approximately 812 MWe during April.

MAJOR SAFETY RELATED MAINTENANCE

The following major safety related maintenance item was accomplished during the month:

Emergency Diesel Generators EG-Y-1A and B

The Emergency Diesel Generators EG-Y-1A and B were removed from service for their scheduled annual overhauls. EG-Y-1B was removed from service during the week of April 15th and EG-Y-1A was removed from service during the week of April 22nd. The work accomplished on both engines and associated equipment was identical: a major engine inspection and overhaul, relief valve testing, oil changes, instrument calibrations, breaker tests, and electrical motor inspections. All work was accomplished with no major discrepancies found on either Emergency Diesel Generators. EG-Y-1A and B were retested satisfactorily and returned to service.

OPERATING DATA REPORT

OPERATING STATUS

DOCKET NO. 50-289
 DATE _____
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

1. UNIT NAME: THREE MILE ISLAND UNIT 1
 2. REPORTING PERIOD: APRIL 1996
 3. LICENSED THERMAL POWER: 2568
 4. NAMEPLATE RATING (GROSS MWe): 872
 5. DESIGN ELECTRICAL RATING (NET MWe): 819
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

THIS MONTH YR-TO-DATE CUMMULATIVE

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	719.0	2903.0	189888.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	719.0	2903.0	112446.1
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	719.0	2903.0	111284.3
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1844646	7448844	273563770
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	617333	2511683	91965157
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	583865	2373361	86116765
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	58.6
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	58.6
21. UNIT CAPACITY FACTOR (USING MDC NET)		103.3	104.0	57.9
22. UNIT CAPACITY FACTOR (USING DER NET)		99.2	99.8	55.6
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	35.2
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: APRIL

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
1	817	17	817
2	818	18	814
3	814	19	812
4	816	20	802
5	818	21	802
6	821	22	798
7	821	23	794
8	821	24	814
9	822	25	809
10	820	26	806
11	813	27	815
12	804	28	813
13	811	29	801
14	816	30	802
15	818	31	#N/A
16	815		

REPORT MONTH April 1996

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code ⁴ & ⁵	Component Code ⁵ & ⁶	Cause & Corrective Action to Prevent Recurrence
						None			

1
F Forced
S Scheduled

2
 Reason
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
 Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
 Exhibit G - Instructions for
 preparation of Data Entry Sheets
 for Licensee Event Report (LER)
 File (NUREG-0161)

5 Exhibit I same source

6 Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 5, 1997**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **Yes. To support GPU Nuclear plans to do independent reload analyses for Cycle 12 as discussed in response to question 6 below, T.S. 6.9.5.2 would require revision to include references to the GPU Nuclear analysis methods applied to the reload consistent with the guidance of Draft Generic Letter (GL) 83-11.**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **A Technical Specification Change Request for the changes as discussed above would be submitted after formal issuance of GL 83-11.**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: **GPU Nuclear Letter 6710-96-2092, dated March 28, 1996 confirmed plans to perform independent reload design evaluations for Cycle 12, the next operation cycle, based on NRC approved methods described in GPU Nuclear Topical Reports TR-091 (core physics), TR-078 (core thermal hydraulics), TR-087 (FSAR safety analyses) and TR-092P (design and setpoints methodology) submitted to the NRC. As submitted in the March 28 letter, Cycle 12 independent reload design activities and application results anticipate the use of formally issued NRC Draft Generic Letter 83-11, Supplement 1, "Licensee Qualification for Performing Safety Analyses". Per the Draft GL 83-11 guidance, the GPU Nuclear Cycle 12 reload program and results are expected to be available for NRC review in the March to April 1997 time frame. It is expected that the Cycle 12 results will be licensed in accordance with the requirements of 10 CFR 50.59 safety evaluations for changes.**
7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) **177** (b) **864**

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained. Phase II is expected to be started in 2002.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.