

May 6, 1996

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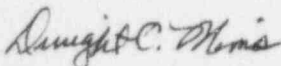
U. S. Nuclear Regulatory Commission
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Washington, DC 20555

Subject: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
Licensee Event Report 50-313/96-004-00

Gentlemen:

In accordance with 10CFR50.73(a)(2)(i)(B), enclosed is the subject report concerning heavy load transport over the spent fuel pool.

Very truly yours,



Dwight C. Mims
Director, Nuclear Safety

DCM/rhs

enclosure

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cc: Mr. Leonard J. Callan
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

Institute of Nuclear Power Operations
700 Galleria Parkway
Atlanta, GA 30339-5957

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Arkansas Nuclear One - Unit 1

DOCKET NUMBER (2)

05000313

PAGE (3)

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TITLE (4) Load In Excess Of Technical Specifications Weight Limit Moved Over Fuel Stored In The Spent Fuel Pool As A Result Of Conflicting Procedural Guidance Which Resulted From An Inadequate Review During Procedure Development

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
03	06	96	96	004	00	05	06	96	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR: (Check one or more) (11)							
POWER LEVEL (10)		100	20.402(b)			20.405(c)			50.73(a)(2)(iv)	70.71(b)
			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)	70.71(c)
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)	OTHER
			20.405(a)(1)(iii)		X	50.73(a)(2)(i)			50.73(a)(2)(viii)(A)	Specify in
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)	Abstract Below
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)	and in Text

LICENSEE CONTACT FOR THIS LER (12)

NAME

Richard H. Scheide, Nuclear Safety and Licensing Specialist

TELEPHONE NUMBER (Include Area Code)

501-858-5000

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES		NO		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
(If yes, complete EXPECTED SUBMISSION DATE)		X					

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 5, 1996, it was identified by ANO personnel that a Technical Specifications violation had occurred on March 6, 1996, when a crane load in excess of 2000 pounds was positioned briefly over fuel stored in the spent fuel pool (SFP). On March 6, ANO personnel were lifting the cask loading pit gate, which weighs approximately 4000 pounds, in preparation for storing it on the edge of the SFP. Due to the presence of steel tabs on top of the gate, it had to be rotated 180 degrees to be stored. As the craft personnel were rotating the gate, a Senior Reactor Operator observed that it was positioned partially over the fuel in the pool. The operator immediately halted the work and directed the craft personnel to reposition the gate and move it to its storage location without passing it over any fuel. The root cause of this event was an inadequate review during procedure development in 1981 which resulted in conflicting guidance indicating that it was acceptable to briefly pass heavy loads over fuel stored in the SFP. Crew briefings were conducted regarding this event and interim controls were established to ensure that an operator is present when the cask loading pit gate is moved pending procedure revision to remove the conflicting information.

NRC FORM 366A (5-92)		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95	
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION				ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.	
FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)	
Arkansas Nuclear One - Unit 1		005000313		YEAR	SEQUENTIAL NUMBER
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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

A. Plant Status

At the time this event occurred, Arkansas Nuclear One Unit-1 (ANO-1) was operating at approximately 100 percent power. Reactor Coolant System (RCS)[AB] temperature was 579 degrees and RCS pressure was 2155 psig.

B. Event Description

On April 5, 1996, it was identified by ANO personnel that a Technical Specifications violation had occurred on March 6, 1996, when a crane load in excess of 2000 pounds was positioned over fuel stored in the Spent Fuel Pool (SFP).

ANO-1 Technical Specification 3.8.14 states, "Loads in excess of 2000 pounds shall be prohibited from travel over fuel assemblies in the storage pool."

The cask loading pit gate, which weighs about 4000 pounds, has two steel tabs on top which may be used to secure the gate in place. These tabs would interfere with the concrete curb around the SFP if the gate were moved directly to its storage position on the side of the pool. Therefore, the gate must be rotated 180 degrees in order to place it in its storage position. The gate is slightly over three feet wide and the distance to the first fuel assembly from the edge of the SFP is approximately 2 1/2 feet.

On March 6, 1996, at approximately 2200, ANO personnel were lifting the cask loading pit gate with the L-3 crane in preparation for storing the gate on the edge of the SFP. As the craft personnel were rotating the gate, a Senior Reactor Operator (SRO) observed that it had been positioned partially over spent fuel which was stored in the fuel racks in the SFP. The SRO immediately stopped the evolution and directed the craft personnel to reposition the gate and move it to its storage location in such a manner as to prevent it from traveling over any fuel in the pool.

On April 5, 1996, a review of the spent fuel crane operating procedure and Technical Specifications identified that a violation of the specifications had occurred on March 6.

C. Root Cause

The immediate cause of this event was determined to be an inadequate procedure which contained conflicting information regarding the movement of SFP gates. The procedure that governs operation of the spent fuel crane (L-3) contains a precaution which states, "Loads in excess of 2000 pounds are prohibited from being moved over the fuel in the SFP's." However, an attachment to the procedure contained a special instruction regarding lifting of the SFP gates which stated, "Avoid bringing gate over spent fuel as much as possible." This guidance, which has been in the procedure since its development in

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1981, could lead crane operators to conclude that it was acceptable to move the gates over fuel for a short time.

The root cause of the event was an inadequate review during development of the procedure which failed to identify that the special instruction contained in the attachment conflicted with the Technical Specifications.

D. Corrective Actions

Pending procedure revision, interim controls were implemented to require an operator to be present whenever the cask loading pit gate is moved.

Crews currently working in the SFP area were instructed in the proper method of rotating the cask loading pit gate.

The spent fuel crane operating procedure was revised to remove the statement that conflicted with the Technical Specifications.

The ANO 10CFR50.59 review process has been vastly improved since 1981, when the error referenced in this report occurred. All 10CFR50.59 reviewers receive formal certification training. The guidelines for review are proceduralized in detail and reviewers have electronic search capability of all licensing basis documents. Therefore, the current 10CFR50.59 review process is considered adequate to prevent the occurrence of similar events and no additional corrective actions are deemed necessary.

E. Safety Significance

The significance of this event is lessened since only a small part (approximately six inches) of the cask loading pit gate was rotated into a position where it was suspended over spent fuel and because the condition was immediately recognized and corrected.

F. Basis for Reportability

ANO-1 Technical Specification 3.8.14 specifically prohibits moving loads weighing over 2000 pounds over fuel stored in the SFP. This specification has no associated action; therefore, this event represents an operation prohibited by the Technical Specifications reportable pursuant to 10CFR50.73(a)(2)(i)(B).

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TEXT CONTINUATION

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G. Additional Information

There have been no previous LERs reported by ANO in which loads exceeding Technical Specifications limits were moved over fuel stored in the SFP.

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].