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Byron Generating Station
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May 8, 1996

LTR: BYRON-96-5081

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Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Attention: Document Control Desk

SUBJECT: Byron Station Unit 1, Steam Generator (SG) Tube Repairs During Byron 1
Refueling Outage (B1R07)

This letter is to inform the staff of SG tubes repaired during the B1R07 inspection conducted between April and May 1996. Repaired conditions are defined as tubes either requiring Inconel-690 mechanical plugs or Inconel-690 sleeves. This information is being reported pursuant to operating License NPF-37, Appendix A, Technical Specification 4.4.5.5a.

All tubes in each SG were inspected from the hot leg tube end to the cold leg tube end for the bobbin eddy current inspection. The following additional inspections were performed during this outage: 100% top of tubesheet (Hot Leg) - Plus Point, 20% top of tubesheet (Cold Leg) - Plus Point - expanding to 100% in SG C, 20% row 1 and row 2 U-Bend Plus Point - expanding to 100% of each SG and 20% in row 3 of SG B, and 20% Preheater expansion region Rotating Pancake Coil in SG B. Based on the results of these inspections, it was determined that each steam generator would be classified as C-3 category pursuant to Technical Specification 4.4.5.2. Byron Station LER 454: 96-003 will be submitted to address the C-3 category classification of the steam generators.

As a result of the above inspections, a total of 3,873 tubes will be repaired, either by plugging the tubes or sleeving the defective region, during the B1R07 outage. The currently expected tube repair distribution is provided below for each steam generator:

Type of Repair (*)	SG A	SG B	SG C	SG D	Totals
1) Plugging	54	54	63	64	235
2) Sleeving	603	860	1050	1125	3638
Total # of Repairs	657	914	1113	1189	3873

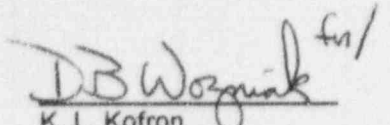
- * Plugging and sleeving of the SG tubes is not complete at this time. Therefore, total numbers for plugging and sleeving may change due to the potential for rejected sleeves. However total number of repairs for each SG will remain unchanged. Any changes made to this repair distribution will be reflected in the 90 day Summary Report to be submitted in accordance with Technical Specification 4.4.5.5.b.

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If there are any additional questions, please contact Mr. Joe Lonigro at (815) 234-5441, extension 2166.


K. L. Kofron
Station Manager
Byron Nuclear Power Station

DBW/cb

cc: H. J. Miller, NRC Regional Administrator -RIII
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Office of Nuclear Safety - IDNS
Site Engineering File